NUCLEAR EXPERIMENT TECHNIQUE

Measurements of the Cumulative Yield of the ¹⁰³Ru Radioisotope through the ¹⁰⁰Mo(⁴He, n + p)¹⁰³Ru Reaction and a Technique for Gas Thermal Separation of ¹⁰³Ru from a ¹⁰⁰MoO₃ Target

V. A. Zagryadskii^a, Ya. M. Kravets^a, T. Yu. Malamut^a, V. I. Novikov^a, A. A. Smirnov^a, T. A. Udalova^{a,*}, and V. N. Unezhev^a

^a Kurchatov Institute National Research Center, Moscow, 123182 Russia *e-mail: udalova_ta@nrcki.ru Received June 1, 2022; revised July 6, 2022; accepted July 8, 2022

Abstract—The cumulative yield of the ¹⁰³Ru radioisotope was measured using the activation method when a ¹⁰⁰MoO₃ target was irradiated by ⁴He nuclei with an energy of 60.3 MeV at the U-150 cyclotron of the Kurchatov Institute National Research Center. The measured ¹⁰³Ru yield was $(4.93 \pm 0.84) \times 10^4$ Bq/(µA h). An experimental technique has been developed for prompt gas thermal separation of ¹⁰³Ru radioisotope from an irradiated target. An experimental setup has been created to implement the ¹⁰³Ru extraction technique. The design of the setup and the principle of its operation are presented. It is shown that the technique ensures the extraction of at least 97% ¹⁰³Ru from the target material and the return of at least 96% ¹⁰⁰MoO₃ for reuse. The developed technique can find practical application in the production of the ¹⁰³Ru radioisotope by irradiating of ¹⁰⁰MoO₃ cyclotron targets with ⁴He nuclei.

DOI: 10.1134/S0020441222060203

INTRODUCTION

A new approach to cancer therapy is currently discussed in a number of publications. This approach is based on the use of modular nanotransporters (MNT) [1]. MNT are synthesized on the basis of a polypeptide platform and are capable of delivering therapeutic "short-range" radionuclides to a cancer cell nucleus. Loading MNTs with "short-range" radionuclides can effectively destroy both individual cancer cells and micrometastases that are not detected by modern diagnostic methods. At the same time, they hardly have a negative cytotoxic effect on healthy cells and tissues. This technique can be used both after surgery and after chemotherapy as a finishing therapeutic procedure aimed at eliminating the cancer recurrence.

It is proposed to use Auger electron emitters in tandem with MNT as therapeutic "short-range" radionuclides [2]. Auger electrons have a short range and a high specific linear energy loss. They are capable of damaging cells within a few tens of nanometers without having a cytotoxic effect at long distances. ^{103m}Rh isotope is considered to be one of the most promising Auger electron emitters for immunotherapy [3]. It has the smallest ratio of the number of emitted γ rays to the number of electrons and can be produced using the generator method. The ¹⁰³Ru radionuclide ($T_{1/2}$ = 39.274 days) is the ^{103m}Rh ($T_{1/2} = 56.1$ min) precursor in the generator. One of the ways to obtain ¹⁰³Ru at the U-150 cyclotron of the Kurchatov Institute National Research Center can be the ¹⁰⁰Mo(⁴He, n + p)¹⁰³Ru reaction. Experimental data on the ¹⁰³Ru cumulative yield in the ¹⁰⁰Mo(⁴He, n + p)¹⁰³Ru reaction are not available in the literature. In order to clarify the efficiency of ¹⁰³Ru production in this reaction, the cumulative yield of ¹⁰³Ru was measured upon irradiating a ¹⁰⁰MoO₃ target by ⁴He nuclei with an energy of 60.3 MeV.

The developed gas-thermal technique was used to extract ¹⁰³Ru from the irradiated ¹⁰⁰MoO₃ target. An experimental setup was manufactured to implement the technique. The structural elements used in the setup and the temperature conditions were selected to ensure the target gasification and almost complete ¹⁰³Ru separation from the ¹⁰⁰MoO₃ matrix.

MEASUREMENT OF THE ¹⁰³Ru CUMULATIVE YIELD IN THE ¹⁰⁰Mo(⁴He, n + p)¹⁰³Ru REACTION

The yield of the ¹⁰³Ru radioisotope was determined using an activation technique. A target in the form of a disk with a diameter of 8 mm and a thickness of 3 mm



Fig. 1. Temperature dependences of the MoO_3 vapor pressure and the partial RuO_3 and RuO_4 pressures over the RuO_2 in the atmosphere O_2 .

was made of MoO₃ powder with molybdenum enriched to 99.7% in the ¹⁰⁰Mo isotope. It was packed into a target device that had a 100-µm-thick aluminum foil window at the beam inlet. The target device was placed in the chamber of the U-150 cyclotron of the Kurchatov Institute National Research Center and was irradiated with ⁴He nuclei at a current of $\approx 0.15 \,\mu\text{A}$ until a total charge of $\approx 0.3 \,\mu\text{A}$ h was attained. The target had a bulk density of 2.0 g/cm³, which was determined by weighing. The energy of the ⁴He nuclei at the entrance to the ¹⁰⁰MoO₃ target was 60.3 MeV. The range of ⁴He nuclei, calculated using the SRIM program [4], fit into the target thickness. The total charge of nuclei incident on the target during irradiation was measured using a special integrating device. The energy of the accelerated nuclei was determined by the cyclotron parameters.

After the target was irradiated and then held for 3 days, the ¹⁰³Ru activity was determined by the fullenergy peak of γ rays with energy $E_{\gamma} = 497.085$ keV ($K_{\gamma} = 91\%$) [5]. The measurements were made using an ORTEC GEM 35P4 γ spectrometer (United States) with a high-purity germanium detector with a volume of ~100 cm³. The targets were placed at a distance of 40 cm from the end surface of the detector during the measurements. The dead time of the spectrometer did not exceed 5% in the activity measurements. The energy dependence of the γ -ray detection efficiency was determined experimentally using reference spectrometric γ -ray sources from an OSGI kit. The acquisition time of γ spectra was 1 h. The target activity was measured several times during the ¹⁰³Ru half-life.

The ¹⁰³Ru yield was determined using the formula

$$V = AZ_{1}[1 - \exp(-\lambda T_{1})] / \{Z_{2}[1 - \exp(-\lambda T_{2})]\}, \quad (1)$$

where V [Bq/(μ A h)] is the ¹⁰³Ru yield; A [Bq] is the ¹⁰³Ru activity in the target, referred to the end of irradiation; Z_1 [rel. units] is the current-integrator reading that corresponds to a charge of 1 μ A h; λ [s⁻¹] is the decay constant of ¹⁰³Ru; T_1 [s] is the irradiation time equal to 1 h; Z_2 [rel. units] is the current-integrator reading over the irradiation time; and T_2 [s] is the the actual time of the target irradiation.

The ¹⁰³Ru cumulative yield obtained in the experiment was $(4.93 \pm 0.84) \times 10^4$ Bq/(µA h). The measurement error of the yield was 17% with a confidence probability of 68%. The following error components were taken into account: the error in determining the detector efficiency, the error in determining the fullenergy peak area in the measured γ -ray spectrum, and the error of the branching fraction.

TECHNIQUE FOR GAS THERMAL SEPARATION OF ¹⁰³Ru FROM A ¹⁰⁰MoO₃ TARGET

The developed technique for gas thermal separation of 103 Ru from a 100 MoO₃ target irradiated at the cyclotron is based on a significant difference in the volatility of MoO₃ and Ru oxides. Figure 1 presents comparative data on the temperature dependence of the MoO₃ vapor pressure [6] and the partial pressures of RuO₃ and RuO₄ [7, 8] over RuO₂ in an oxygen atmosphere. According to Fig. 1, the partial pressures of MoO₃ vapors and Ru oxides in comparable temperature ranges differ by several orders of magnitude.

A setup has been created to implement the technique for separating ¹⁰⁰MoO₃ and Ru oxides; its diagram is shown in Fig. 2. The setup was evacuated by forevacuum pump 10 and was filled with oxygen from cylinder 1 using reducer 16. The central elements of the setup were two coaxial fused-silica ampoules 4 and 5, which were connected to a gas stand, and crucible 7 with irradiated ¹⁰⁰MoO₃ powder. The external fusedsilica ampoule 4 was connected to the gas stand using a vacuum seal through a fitting made of D16 aluminum alloy. One end of the fitting was glued to the ampoule by ED-6 epoxy resin, and the other was connected to the flanges of the stand using gaskets. Such a device simplifies the sealing process, since it allows less stringent requirements for the quality of the fusedsilica ampoule (the degree of its ellipsis and the surface roughness) and reduces the risk of ampoule splitting during the sealing. The vacuum or the required O_2



Fig. 2. Block diagram of the experimental setup for separating ¹⁰³Ru from the cyclotron ¹⁰⁰MoO₃ target: (*I*) oxygen cylinder, (*2*) AIR-20-M2-DA pressure gauge, (*3*) 1.9-L measuring flask, (*4*) external fused-silica ampoule ($D_{out} = 35.5 \text{ mm}$, $d_{in} = 31 \text{ mm}$, l = 400 mm), (*5*) two replaceable internal fused-silica ampoules ($D_{out} = 27 \text{ mm}$, $d_{in} = 22.5 \text{ mm}$, l = 250 mm), (*6*) thermostat-heater, (*7*) fused-silica crucible ($D_{out} = 15 \text{ mm}$, $d_{in} = 12 \text{ mm}$, l = 17 mm) with irradiated ¹⁰⁰MoO₃ powder containing ¹⁰³Ru, (*8*) chromel–alumel thermocouple, (*9*) IRT/M2 temperature meter, (*10*) NVR-5D forevacuum pump, (*11–15*) valves, and (*16*) GSE gas reducer.

pressure in ampoule 4 was maintained at various temperature conditions provided by thermostat-heater 6.

While testing the technique, we noticed that a black deposit of ${}^{100}\text{MoO}_2$ appeared on the walls of the crucible after vacuum sublimation of ${}^{100}\text{MoO}_3$ at 700°C, and its weight was $\approx 1\%$ of the initial ${}^{100}\text{MoO}_3$ weight.

According to [9, 10], the appearance of MoO₂ was caused by the content of a certain proportion of lower oxides (Mo₂O₅, Mo₄O₁₁, Mo₅O₁₄) in MoO₃. When MoO₃ sublimates in vacuum, they disproportionate according to the schemes Mo₂O₅(solid) \leftrightarrow MoO₃(gas) + MoO₂(solid) and Mo₄O₁₁(solid) \leftrightarrow (MoO₃)₃(gas) + MoO₂(solid), thus leaving MoO₂ in a solid state under these conditions.

Therefore, after removing the bulk of $^{100}MoO_3$, we needed an additional technological operation to gasify the residual $^{100}MoO_2$.

In view of the above, the developed ¹⁰³Ru separation technique consisted of a sequence of three technological operations implemented in three stages. Figure 3 illustrates the results obtained at each of the three stages.

At the first stage, ${}^{100}MoO_3$ was distilled in vacuum at a temperature of 700°C. To do this, irradiated ${}^{100}MoO_3$ powder with a mass of 113 mg was loaded into crucible 7. After that, the crucible was inserted into inner ampoule 5, which in turn was placed in outer ampoule 4. Ampoule 4 was sealed and then connected to the vacuum stand upon immersing in thermostat-heater 6 to a depth of 10 cm. Evacuation to a pressure of $\approx 2 \times 10^{-2}$ mm Hg was carried out using forevacuum pump 10. A voltage was then applied to the thermostat-heater, the temperature was raised to 700°C for 45 min, and this temperature was maintained for 20 min. the ¹⁰⁰MoO₃ sample was sublimated and deposited on the walls of ampoule 5. At the end of the sublimation process, the thermostat-heater was removed, the system was cooled, and the inner ampoule and crucible were withdrawn. The crucible and ampoule 5 were weighed to balance the masses. Monitoring of the ¹⁰³Ru activity in the crucible before and after the technological operation was carried out by the relative intensity of the 497.08-keV γ -ray peak of 103 Ru. It was stated that at least 99% of the 100 MoO₃ mass were removed from the crucible at the first stage, but the ¹⁰³Ru activity in the crucible did not change. However, a smoky black coating of ¹⁰⁰MoO₂ remained on the crucible walls (see Fig. 3, stage 1).

The ${}^{100}\text{MoO}_2$ remaining in the crucible was oxidized to ${}^{100}\text{MoO}_3$ at the second stage. To do this, the crucible was placed in a clean internal ampoule, and the preparatory operations described above were performed. The entire system (with open valves 12-15)

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Fig. 3. Illustration for the results of the phased 103 Ru separation from 100 MoO₃.

was pumped down to a pressure of $\approx 2 \times 10^{-2}$ mm Hg. Valve 15 was then closed, and the entire system was filled with oxygen from cylinder 1 using reducer 16 to a pressure of 75 kPa, which was monitored by pressure gauge 2. Next, valve 12 was closed, a voltage was applied to the thermostat-heater, the temperature was raised to 475°C, and this value was maintained for 20 min. After ampoule 4 was cooled, the crucible was removed, the intensity of the 497.08-keV γ -ray peak of ¹⁰³Ru was measured by the area of the full-energy peak, the crucible was weighed, and the mass of molybdenum oxides contained in it was determined. As a result of oxidation (see Fig. 3, stage 2), the smoky black coating of ¹⁰⁰MoO₂ on the walls of the crucible acquired a light gray color that is characteristic of ¹⁰⁰MoO₃. (The mass change during this operation was within the measurement error.) It was found that the relative decrease in the activity of ¹⁰³Ru in the crucible after the oxidation procedure, which was determined by the intensity of the 497.08-keV γ -ray peak of ¹⁰³Ru, did not exceed 3%.

At the third stage that was similar to stage 1, $^{100}MoO_3$ residues (i.e., the oxidized $^{100}MoO_2$) were sublimated from the crucible in a vacuum at a temperature of 700°C. The 103 Ru activity in the crucible did not change, and the walls of the fused-silica crucible became transparent (see Fig. 3, stage 3).

To close the circulation cycle of the ¹⁰⁰Mo raw isotope, the ¹⁰⁰MoO₃ deposit on the walls of ampoule 5 was washed off with a 10% solution of ammonium hydroxide (NH₄OH). As a result of the ¹⁰⁰MoO₃ interaction with ammonium hydroxide, ammonium paramolybdate (NH₄)₆¹⁰⁰Mo₇O₂₄ was formed in the solution. By evaporating the solution and calcining the ammonium paramolybdate deposit, ¹⁰⁰MoO₃ was obtained from the reaction (NH₄)₆¹⁰⁰Mo₇O₂₄ = $6NH_3^{\uparrow} + 7^{100}MoO_3 + {}^{3}H_2O^{\uparrow}$ according to the technique [11, 12]. The ¹⁰⁰MoO₃ loss during ammonium paramolybdate calcination did not exceed 4%.

CONCLUSIONS

The cumulative yield of the ¹⁰³Ru radioisotope was measured for the first time using the activation method. The ¹⁰⁰MoO₃ target was irradiated by ⁴He nuclei with an energy of 60.3 MeV. The value of the measured ¹⁰³Ru yield was $(4.93 \pm 0.84) \times 10^4$ Bq/(µA h). The experimental technique was developed for express gas thermal separation of ¹⁰³Ru radioisotope from the irradiated ¹⁰⁰MoO₃ cyclotron target. The technique ensured the extraction of at least 97% of ¹⁰³Ru from the target material and the return of at least 96% ¹⁰⁰MoO₃ for reuse.

FUNDING

This work was supported by the Kurchatov Institute National Research Center, order no. 2751 dated October 28, 2021.

CONFLICT OF INTEREST

The authors declare that they have no conflicts of interest.

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Translated by N. Goryacheva