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Exploring Gamma Radiation Shielding: the Role of BaO in Borosilicate Glasses

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Abstract

The gamma ray shielding characteristics of different borosilicate glasses are examined in this work. Four glasses with a composition of $60B_2O_3$ – $5Na_2O$ –5PbO– $(30-x)SiO_2$ –xBaO, (x = 5, 10, 15, and 20 mol%) were created using the conventional melt quenching technique followed by an annealing step. Linear attenuation coefficient, LAC, values have been determined using HPGe semi-conductor detector. These values were compared with calculated values estimated from Phy-X software and a good matching was observed. The samples were irradiated using the point sources viz., Am²⁴¹ (0.0595 MeV), Cs¹³⁷ (0.6617 MeV) and Co⁶⁰ (1.173 and 1.330 MeV). The LAC data were further utilized in computations of other radiological parameters that are half value layer (HVL) and Tenth value layer (TVL). Furthermore, radiation shielding efficiency (RSE) of the prepared glass materials has been evaluated. The sample 10S20B exhibits higher values of LAC than the others because it has the largest density and weight fraction of elements with higher atomic numbers. The significance of the atomic number and density imply greater probability of interaction, leading to better attenuation.

Keywords Gamma-rays · HPGe-detector · Attenuation · Shielding · Mean free path

1 Introduction

Gamma radiation, known as a combination of electric and magnetic radiation, is identified by super-energy photons originating from nuclei of an atom in the course of nuclear processes. Its existence is permeating in copious areas,

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apparently in medical surroundings where therapeutic and diagnostic policies subjects' humans to these influential rays [1-3]. As favorable as these uses may be, lengthen or immoderate subjection to gamma beam is accompanied with critical health danger, counting tissue injury, DNA damage, and an upraised cancer risk [4-6].

In the hospital's empire, gamma beam participates a decisive part in treatment of cancer and medical imaging. Diagnostic policies such as computed tomography (CT) and X-rays scans make use of gamma beam to envisage inner structures, abet in the diagnosis of profuse medical situations. In therapeutic implementations, gamma beam is adopted in radiation therapy in order to prey and demolish cancer cells [7–9]. While these policies have transfigured medical operations, it is imperious to confess and alleviate the effect anguish caused by gamma ray to both healthcare executives and sick persons [10–12].

The injurious consequence of gamma radiation in hospital empire are multitudinous. Lengthen subjection to gamma beam can cause damage to DNA, resulting to alterations that may donate to the cancer occurrence. More so, gamma ray can pose injury to tissue, specifically in subtle organs or fast dividing cells [13–15]. The effects are intensified for healthcare personnel that are regularly

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subjected in the course of administering the therapeutic and diagnostic policies. In the background of the treatments of cancer, while the target is to hit infectious cells, enveloping tissues that are healthy might also be tempered, necessitating an intricated equilibrium inbetween therapeutic effectiveness and reducing surety damages [16–18].

The necessity of successful shielding in opposition to gamma rays appear from the imperious to protect the general public from its damaging implications [19, 20]. Shielding materials refers to barriers that scatter, attenuate or absorb gamma beam, denying them from piercing into living tissues. Any material's ability to provide shielding depended on a number of factors, including its atomic number, density, and composition [21–23]. Conventional materials like concrete, steel and lead have been broadly adopted for shielding of gamma radiation because of their intrinsic characters. Lead, having super atomic number, is exceptionally essential in gamma radiation absorption [24–26].

Nonetheless, evolution in material science have resulted in the inspection of substitute substances that gives distinctive upper hand for shielding of gamma radiation. Oxide of sodium (Na₂O), trioxide of boron (B_2O_3), oxide of barium (BaO), oxide of lead (PbO) and dioxide of silicon (SiO₂) are part of the materials receiving attentiveness for their prospectiveness in this realm [27–31].

One of the major factors that determined the effectiveness of shielding is density, as it control the tendency of gamma beam interrelate with the material. Super densed materials allow more chances for gamma beam to be soak up or dispersed, lowering their capability to pierce via matter [32–34].

Oxide of lead (PbO) is selected because of its superime atomic number. The atomic number is a key factor of a material's capability to suck up gamma radiation. Materials with superior atomic numbers are highly essential in absorption of gamma beam, and oxide of lead, with its upraised atomic number, is excellent in this view [35–39].

Dioxide of silicon (SiO_2) contributes a structural character in shielding of gamma radiation. Even though, it is less dense compare to lead, SiO_2 donates to the entire reliability and solidity of all materials used for shielding applications [40–45]. Supporting structurally is very paramount, particularly in cases where the materials for shielding are required to retain their original form and productiveness with time [46, 47].

Oxide of barium (BaO) improves shielding of gamma radiation by enlarging the density of the material. The character of density can't be exaggerated in shielding of gamma radiation, and BaO's donation in this regard buttresses the entire efficacy of materials used for shielding applications [48, 49].

Introduction of B_2O_3 , Na_2O , PbO, SiO₂, and BaO in shielding of gamma radiation does not only end in advancement technology; it is a requirement to improve protection protocols in medical arena [50–52]. As technology in medicine advances and policies become highly advanced, the call for accurate and systematic shielding of gamma radiation escalated. The distinctive character of such materials, when merged together, present a specialized and modern cure that excel the abilities of conventional materials used for shielding application [53–55].

In the active terrain of healthcare, where the interest of gamma radiation is required, it is pivotal to put in front the protection of healthcare personnel and sick persons. Building in modern materials for shielding is considered as a dedication to developing advancements in technology, making sure that the upper hands of medical policies are maximal while the related consequences are minimal [53, 56, 57].

This study will therefore investigate the gamma radiation shielding ability of B_2O_3 -Na₂O-PbO-SiO₂-BaO with BaO replacing SiO₂ in different percentages.

2 Materials and Methods

2.1 Glasses Preparation

Using the traditional melt quenching procedure followed by an annealing process, four glasses in the series were created, each with the following composition: $60B_2O_3-5Na_2O-5PbO-(30-x)SiO_{2-x}BaO$, where x = 5, 10, 15, and 20 mol%. Table 1 provides the chemical composition for the four prepared glasses. To ensure that every glass composition had a uniform and homogenous structure, the powders were carefully mixed after all the chemicals that would be added to each glass composition were measured precisely. Using a high-purity alumina crucible, the powders were added to an electric oven and heated to 1100 °C for 3 h. The glass was carefully poured into a stainless steel mold once it had completely liquefied. To lower the internal tension, the glasses underwent an annealing process in a different furnace, where they were heated to 400 °C for five hours. In Figs. 1 and 2 we exhibit the schematic process for

 Table 1
 Chemical formula and the density (g/cm³) of fabricated samples

Code	Chemical formula	Density (g/cm ³)
25S5B	60B ₂ O ₃ - 5Na ₂ O-5PbO-25SiO ₂ -5BaO	3.0058
20S10B	60B ₂ O ₃ -5Na ₂ O-5PbO-20SiO ₂ -10BaO	3.2001
15S15B	60B ₂ O ₃ -5Na ₂ O-5PbO-15SiO ₂ -15BaO	3.3962
10S20B	60B ₂ O ₃ -5Na ₂ O-5PbO-10SiO ₂ -20BaO	3.5942

the fabrication of the glasses and a photograph of the producing glasses, respectively.

2.2 Experimental Attenuation Measurements

The attenuation coefficients were determined using the experimental technique by a high pure germanium semiconductor detector with a relative efficiency of 24% and it was used to detect gamma lines emitted from CO-60, Cs-137 and Am-241 sources. The energy range covered by these sources is 0.06–1.33 MeV. The HPGe detector and the gamma source were placed in front of a lead collimator as part of the narrow beam method measurements. Figure 3 shows a schematic of the experimental setup for attenuation factor measurements. The present glass composite is added between the HPGe-detector and

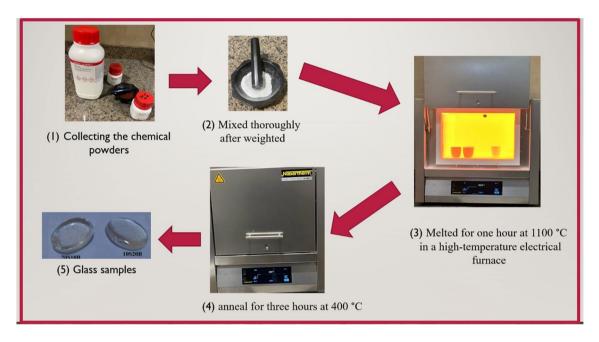
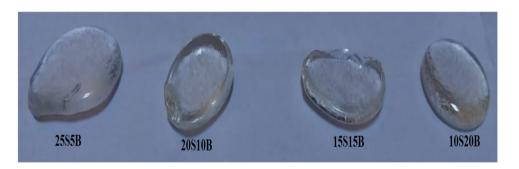
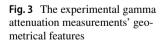
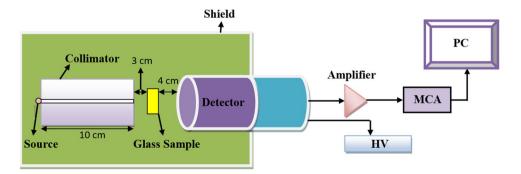


Fig. 1 Schematic process for the fabrication glasses

Fig. 2 Photograph of the producing glasses







the gamma-ray source at an appropriate location. The net counting rate of the gamma line recorded in the glass composite present (N) and the other in the absence of the glass sample (N0) was determined. From these values, the experimental linear attenuation coefficient (LAC, cm^{-1}) can be determined by the following formula [58–61]:

$$LAC, cm^{-1} = \frac{1}{t} \ln \frac{N_0}{N}$$

$$\tag{1}$$

where the thickness of the placed glass composite is denoted by t. The following laws can be used to express and calculate the other significant shielding factors, such as HVL, MFP, TVL, and Radiation shielding efficiency (RSE), based on the determination of N and N0. [62–64]:

$$HVL = \frac{Ln(2)}{\mu} \tag{2}$$

$$MFP = \frac{1}{\mu} \tag{3}$$

$$TVL = \frac{Ln(10)}{\mu} \tag{4}$$

$$RSE, \% = [1 - \frac{N_0}{N}] \times 100$$
(5)

3 Result and Discussion

Table 1 shows the specifics of the samples produced in terms of their densities and chemical contents. Linear attenuation coefficient, LAC, values have been determined using HPGe detector. The samples were irradiated using the point sources viz., Am^{241} (0.0595 MeV), Cs^{137} (0.6617 MeV) and Co^{60} (1.173 and 1.330 MeV). The results thus obtained were confirmed using the well-known Phy-X program. LAC data were further utilized in computations of other radiological parameters that are

HVL and MFP. Furthermore, radiation shielding efficiency (RSE) of the prepared glass materials has been evaluated.

Table 2 depicts LAC values derived from experiments and Phy-X program for the samples along with the differences between them. As seen, relative deviations between LAC values derived from experiments and Phy-X program are minuscule. To give an instance, for 25S5B glass at low energy 0.05595 MeV, experimental value 4.4790 is validated by Phy-X value which is 4.7752. Also, at higher energy (1.330 MeV) for the 25S5B glass, experimental value 0.1566 is confirmed by Phy-X value of 0.1621. The deviation in experimental and theoretical values for the studied glass samples are in the range of 0.76–7.41. From Fig. 4, it is clear from this that LAC is dependent on incident energy and the chemical composition of the samples. It displays the change in LAC values of the chosen glass samples over the photon energy range of 0.0595 MeV to 1.330 MeV. The sample 10S20B exhibits higher values of LAC than the others because it has the largest density and weight fraction of elements with higher atomic numbers. The significance of the atomic number and density parameters-higher atomic number and density imply greater probability of interaction, leading to better attenuation-can be further explained by the described research by El-Khatib et al. and More et al. [65–67]. A sharp decrease is observed in LAC values from 0.0595 to 0.6617 MeV and after that low discrepancy is observed. The dominance of the photoelectric effect at lower energy in which interaction cross section relies on energy as $\sigma Ph \sim E - 7/2$ and on the atomic number (Zn), where n varies from 4 to 5 determine the probability of interaction. Compton effect (σ Com ~ E-1) dominates at intermediate energies. The LAC values showed that all of the samples had the identical attenuation levels at these energies. This can be attributed to the linear dependency of Compton scattering on atomic number, Z [68, 69]. For instance, 25S5B and 20S10B have LAC values of 0.2422 and 0.2574 sequentially at 0.6617 MeV. The difference between these values is of 0.015 cm-1 only. Figure 5 shows comparison of LAC values with the materials available in the literature at 0.0595 and 0.6617 MeV. It is clearly noted that, 10S20B is better gamma ray attenuator than bismuth oxychloride-filled polyester concretes; hematite doped polymer composites; composites high density polyethylene (HDPE) with PbO and

Table 2 The LAC values derived from experiments and Phy-X program for glass samples along with the differences between them

	Glass sample	0.0595 MeV		0.6617 MeV		1.173 MeV			1.333 MeV				
		Phy-X	Exp	Dev (%)	Phy-X	Exp	Dev (%)	Phy-X	Exp	Dev (%)	Phy-X	Exp	Dev (%)
LAC, cm ⁻¹	25S5B	4.7752	4.4790	6.61	0.2422	0.2365	2.44	0.1739	0.1662	4.64	0.1621	0.1566	3.54
	20S10B	7.0654	6.6909	5.60	0.2574	0.2464	4.48	0.1836	0.1754	4.71	0.1712	0.1636	4.59
	15S15B	9.3786	9.0304	3.86	0.2727	0.2611	4.43	0.1935	0.1875	3.16	0.1803	0.1737	3.79
	10S20B	11.7154	11.4031	2.74	0.2881	0.2683	7.41	0.2034	0.2019	0.76	0.1895	0.1821	4.08

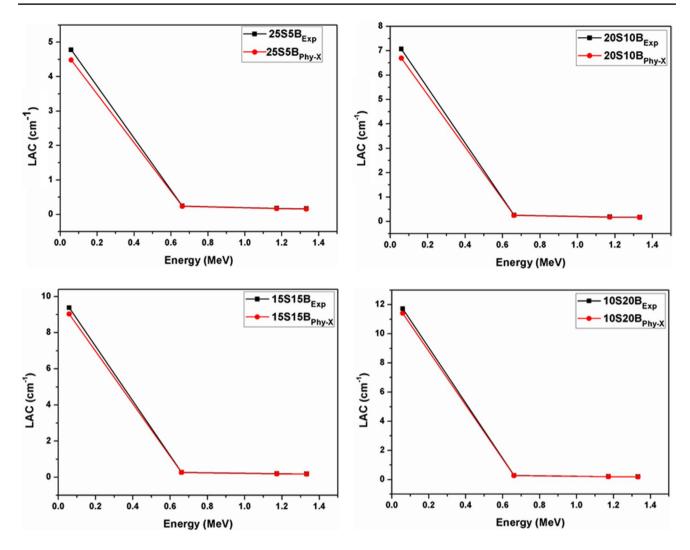


Fig. 4 Variation of LAC values of the tested samples as a function of incident photon energy

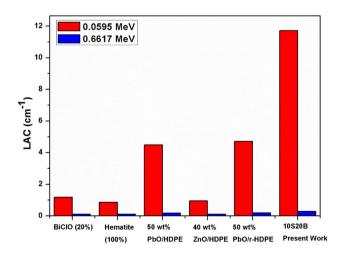


Fig. 5 LAC values compared with the materials available in the literature at 0.0595 and 0.6617 MeV [5–9]

ZnO nanoparticles as well as composites of recycled HDPE with PbO nanoparticles. Figures 6 and 7 show comparison of LAC values with silica-based commercial glasses at 0.0595 MeV and borosilicate glasses at 0.6617 MeV respectively. It is evident that, 10S20B glass sample from our present work has better gamma ray absorption capabilities.

Figures 8, 9 and 10 display how the MFP, TVL, and HVL change with incident photon energy. These are important characteristics that provide the necessary material thicknesses at certain energy and the material's ability to shield. These variables exhibit the exact opposite trend of variation from LAC, that is, an increasing trend with incident energy. At 0.0595 MeV, values of these parameters are lowest for all the samples. These thicknesses are in the range 0.059–0.145 cm (HVL); 0.085–0.209 cm (MFP); 0.197–0.482 cm (TVL) at lower energy. Suggesting that very thin layer of sample is required to shield photons. Among the chosen samples, values of these parameters for 10S20B

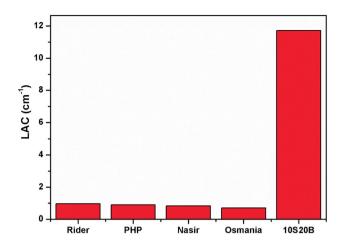


Fig.6 LAC values of 10S20B (present work) compared with silicabased commercial glasses at 0.0595 MeV

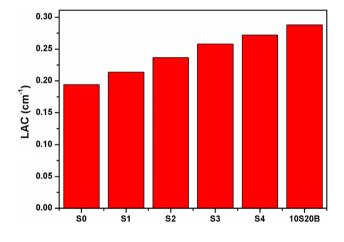


Fig. 7 LAC values of 10S20B (present work) compared with borosilicate glasses at 0.6617 MeV

< 15S15B < 20S10B < 25S5B resulting in 10S20B as better radiation shield among the glasses under study. This is explained by the materials' maximum density relative to all those examined that helps in minimizing values HVL, TVL, and MFP that raised the likelihood of interaction for the 10S20B material. These findings are consistent with earlier research [70–75].

The effectiveness of a shielding material is determined by several parameters, one of the crucial parameters is its radiation protection efficiency, RSE, and RSE (%) of the investigated glasses as a function of energy have been portrayed in Fig. 11. An inverse relation is clearly observed between energy and RSE [76–79]. This declining tendency is brought on by higher energy photons' greater penetrating power, which lessens ability of these glasses to absorb/block incoming radiation. At 0.0595 MeV, 25S5B glass has RSE 85% and other glasses have RSE in order of 95–99% indicating that the

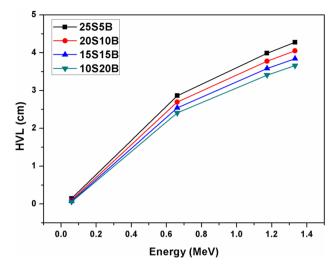


Fig.8 Variation of HVL values of the selected glass samples as a function of incident photon energy

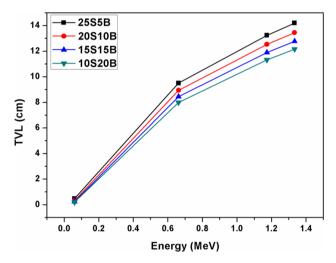


Fig. 9 Variation of TVL of the selected glass samples as a function of incident photon energy

glasses under study are very good at attenuating the lowerenergy photons. Among the selected glasses, 10S20B glass has shown greatest radiation shielding efficiency.

4 Conclusion

Various new compositions of borosilicate glass based on replacing part of SiO_2 with BaO were prepared and their shielding efficiency against gamma rays with different energies was studied. It was concluded that replacing part of SiO_2 with BaO improves the attenuation properties of the proposed glass, and as the percentage of BaO increases, the LAC increases with all the energies studied. The sample

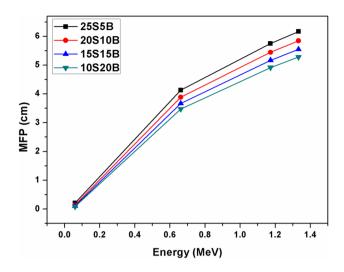


Fig. 10 Variation of MFP of the selected glass samples as a function of incident photon energy

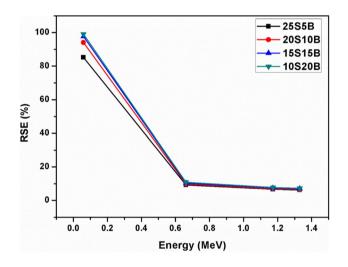


Fig. 11 Variation of RSE values of the selected glass samples over the photon energy from 0.060 MeV to 1.330 MeV

10S20B exhibits higher values of LAC than the others because it has the largest density and weight fraction of elements with higher atomic numbers. The significance of the atomic number and density parameters-higher atomic number and density imply greater probability of interaction, leading to better attenuation. The results of shielding values have been obtained experimentally and compared with Phy-X results and a good matching was observed. An inverse relation is clearly observed between energy and the radiation shielding efficiency, while the 25S5B glass sample has lower RSE and 10S20B glass sample has higher RSE in this work.

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Data Availability No datasets were generated or analysed during the current study.

Declarations

Ethics Approval Not applicable.

Consent to Participate Not applicable.

Consent for Publication Not applicable.

Competing Interests The authors declare no competing interests.

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