



# Radiation Attenuation Properties of Zinc-Borosilicate Glasses Containing $\text{Al}_2\text{O}_3$ and $\text{Gd}_2\text{O}_3$

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## Abstract

This study explores the radiation shielding properties of an  $x\text{Gd}_2\text{O}_3 - (60-x)\text{ZnO} - 15\text{B}_2\text{O}_3 - 5\text{Al}_2\text{O}_3 - 20\text{SiO}_2$  glass system ( $x=0-3$  mol %) glass system using FLUKA and XCOM software. The glass system is interesting due to its potential applications in nuclear and medical settings, where effective radiation-shielding materials are crucial. The FLUKA Monte Carlo simulation package is utilized to model the interaction of gamma photons with the glass system. At the same time, the XCOM software calculates the macroscopic cross-sections based on their elemental composition. The investigation evaluates parameters such as attenuation coefficients, dose rates, neutron cross-section, and stopping power for different radiation sources and energies. Various configurations are analyzed by varying the  $\text{Gd}_2\text{O}_3$  concentrations in the present glass system to optimize shielding effectiveness. The results refer to the high radiation shielding potential of gamma rays and neutrons for the present glassy materials compared to previously studied shielding materials. The present study offers valuable insights into the radiation shielding properties of the studied glass system, facilitating the design and development of efficient radiation shielding materials for practical implementation in nuclear and medical industries.

**Keywords** Borosilicate Glass · Radiation attenuation · Shielding application · FLUKA

## 1 Introduction

Nuclear science finds extensive application in various sectors, including nuclear facilities, research reactors, agriculture, industry, and nuclear medicine. These diverse applications result in the emission of different types of radiation, which can pose risks to the environment, animals, and people. Consequently, numerous researchers

are actively discovering improved materials for radiation attenuation and shielding purposes. The primary objective is to develop materials that effectively mitigate the harmful effects of radiation by acting as barriers between radioactive sources and the surrounding area or workers, thereby reducing exposure to potentially dangerous radiation.

Radiation and nuclear medicine are just a few technological fields where radioactive isotopes are finding more appealing applications today [1, 2]. The environment, animals, and people can all be harmed by radiation gamma, X-rays, and neutrons. Shielding material is well recognized for providing a barrier between a radioactive source and the surrounding area or workers to decrease exposure to hazardous radiation [3]. So, the importance of radiation-shielding materials has increased. Many authors are interested in discovering new glassy systems to use as promised radiation shielding materials [4–8]. This knowledge is applied to produce new materials with specific traits and characteristics suitable for various technological purposes [5–9]. It is generally accomplished using dense materials such as lead or concrete that obstruct or deflect radiation [10]. Other materials, such as boron, are used for neutron shielding [6]. Also, cement, gypsum, borate glass, polymers, and lead are being used for radiation

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shielding purposes, either conventionally or in complex ways [11–13]. Several researchers have discussed numerous systems for experimental measurements and theoretical work using various software on how radiation interacts with these materials (e.g., FLUKA, MCNP, Phy-X, XCOM, and GEANT4) [14–19]. The need for a reduction in exposure rate, source type, space restrictions, and ultimate cost-effective analysis influences the selection of materials. For example, Al-Buriah et al. studied the gamma-ray shielding performance of a  $\text{MoO}_3\text{-TeO}_2$  glass system that showed a promising application in nuclear safety, whether in nuclear or medicine facilities [20]. Gokhan et al. synthesized Ytterbium (III) oxide-reinforced novel  $\text{TeO}_2$ -based glasses that are investigated through their optical, structural, physical, and thermal properties and their radiation shielding ability [21]. Moreover, different simulation codes were used to assess the linear attenuation coefficient, which is a major part to calculate the other gamma shielding parameters such as HVL, MFP, Zeff, etc. [22–25].

Recent research has investigated the influence of  $\text{Gd}_2\text{O}_3$  on the elastic properties of zinc borotellurite glass [26]. The findings suggest that the elastic moduli of these glassy materials increase with increasing  $\text{Gd}_2\text{O}_3$ , leading to improved connectivity and compactness of the glass network. This enhanced connectivity is attributed to the solidification effects of  $\text{Gd}_2\text{O}_3$ . In a related study, Morshidy et al. [27] explored the incorporation of  $\text{Gd}_2\text{O}_3$  as a co-dopant in a chromium borate glass system. The researchers observed that introducing  $\text{Gd}_2\text{O}_3$  resulted in increased compactness of the glass network, as evidenced by higher experimental density measurements. The presence of  $\text{Gd}_2\text{O}_3$  further reinforced the structural integrity of the glass, leading to improved mechanical properties. These investigations demonstrate the positive influence of  $\text{Gd}_2\text{O}_3$  on the elastic ability and compactness of zinc boro-tellurite glass and chromium borate glasses. These findings contribute to understanding how  $\text{Gd}_2\text{O}_3$  can be utilized to enhance the structural properties of glass materials, potentially opening up new possibilities for tailored applications in various fields.

Similarly, the scientific community has also shown keen interest in investigating the effect of  $\text{Gd}_2\text{O}_3$  on the radiation shielding performance of  $\text{TeO}_2\text{-ZnO-Nb}_2\text{O}_5$  glasses [28] and  $\text{Sb}_2\text{O}_3\text{-PbO-B}_2\text{O}_3$  [29]. However, no study shows the impact of  $\text{Gd}_2\text{O}_3$  on the radiation/gamma shielding ability of  $\text{ZnO-5Al}_2\text{O}_3\text{-15B}_2\text{O}_3\text{-20SiO}_2$  glass system, which can be considered one of the most applicable glasses. Therefore, this point encouraged the authors to establish the present work to pave the way for developing advanced glass materials with enhanced performance for various practical applications.

## 2 Materials and Method

### 2.1 Materials

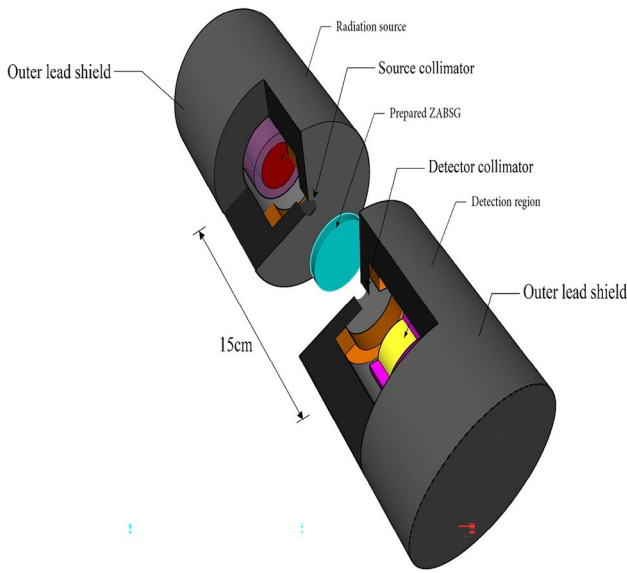
The ternary  $x\text{Gd}_2\text{O}_3 - (60-x)\text{ZnO-15B}_2\text{O}_3\text{-5Al}_2\text{O}_3\text{-20SiO}_2$  glass system ( $x=0\text{--}3$  mol %) with a density range of  $3.775\text{--}4.109\text{ g.cm}^{-3}$  is deliberated to be tested using  $\gamma$ -rays, fast and thermal neutrons using the radiation shielding parameters (e.g.,  $\mu/\rho$ ,  $\mu$ , etc.). These investigated glass systems have been prepared as described in [30]. The elemental chemical compositions and densities of the prepared glass samples are specified in Table 1.

### 2.2 FLUKA Simulation

The radiation shielding simulations of the investigated glasses were conducted using the FLUKA Monte Carlo code. A mono-energetic point source with an energy range of  $0.015\text{--}15$  MeV was employed to simulate the transit of electrons, neutrons, and gamma photons, considering the physics interaction principles such as photoelectric (PEEI), Compton scattering (Com), and pair production processes (PP). To perform the simulations, precise data regarding source dimensions, source-to-detector distance, geometry, elemental chemical composition, and other parameters were incorporated into the FLUKA input files, as shown in Fig. 1. The experimental system encompassed six components: a radioactive source, primary gamma-ray collimator, cylinder

**Table 1** Sample code and weight fraction (wt.%) of element present the prepared  $\text{ZnO-Al}_2\text{O}_3\text{-B}_2\text{O}_3\text{-SiO}_2\text{-Gd}_2\text{O}_3$  glasses including their measured density

Sample	Chemical composition (mol%)	Composition in wt.%						Density ( $\text{g/cm}^3$ )
		B	O	Al	Si	Zn	Gd	
ZABSG1	$60.0\text{ZnO-5Al}_2\text{O}_3\text{-15B}_2\text{O}_3\text{-20SiO}_2$	4.24562	33.51028	3.53200	7.35303	51.35905	0.00000	3.775
ZABSG2	$59.5\text{ZnO-5Al}_2\text{O}_3\text{-15B}_2\text{O}_3\text{-20SiO}_2\text{-0.5Gd}_2\text{O}_3$	4.16892	33.11052	3.46819	7.22019	50.01090	2.02128	3.860
ZABSG3	$59.0\text{ZnO-5Al}_2\text{O}_3\text{-15B}_2\text{O}_3\text{-20SiO}_2\text{-1.0Gd}_2\text{O}_3$	4.09494	32.72494	3.40664	7.09206	48.71060	3.97083	3.909
ZABSG4	$58.0\text{ZnO-5Al}_2\text{O}_3\text{-15B}_2\text{O}_3\text{-20SiO}_2\text{-2.0Gd}_2\text{O}_3$	3.95458	31.99343	3.28988	6.84897	46.24370	7.66944	4.011
ZABSG5	$57.0\text{ZnO-5Al}_2\text{O}_3\text{-15B}_2\text{O}_3\text{-20SiO}_2\text{-3.0Gd}_2\text{O}_3$	3.82353	31.31040	3.18085	6.62200	43.94030	11.12292	4.109



**Fig. 1** FLUKA simulation setup used for evaluating the gamma-ray transmission parameters of the prepared ZABSG<sub>x</sub> glasses

sample, secondary gamma-ray collimator, detector, and an outer lead shield. The radioactive source was positioned within a lead collimator for primary gamma rays, with a radius of 0.14 cm and a height of 0.3 cm. It was located 15 cm away from the detector. A monoenergetic gamma-ray point source was defined for each input file in the energy range of 0.015–15 MeV. The samples under investigation were represented as cubic layers between the source and the detector. The material card contained information regarding the elemental composition and densities of the investigated  $x\text{Gd}_2\text{O}_3 - (60-x)\text{ZnO} - 15\text{B}_2\text{O}_3 - 5\text{Al}_2\text{O}_3 - 20\text{SiO}_2$  glass samples. The detector was configured within a lead collimator for secondary gamma rays. The simulations were conducted with an outer lead shield surrounding the detector, source, collimators, and samples.

**2.3 XCOM Software**

The X-Com software is a web-based program that calculates many different factors associated with shielding and attenuation for the compositions of materials and dosimetry that have been investigated [31]. The mass attenuation coefficients ( $\mu\text{m}$ ) were among the many numbers calculated from the XCOM input file. The input parameters include the elemental makeup and the energy ranges. To determine the relative differences (Dev., %) between the XCOM and FLUKA results,  $\text{Dev. (\%)} = (\text{FLUKA} - \text{XCOM}) / (\text{FLUKA}) \times 100$ .

**2.4 Gamma Ray's Attenuation**

Shielding materials can be identified by measuring their mass attenuation coefficient, which can be determined via the Lambert–Beer's law equation as follows [29]:

$$I = I_0 e^{-\mu x}, \tag{1}$$

Where  $I$  is the intensity of transmitted gamma through the material,  $I_0$  is the intensity of primary gamma in air,  $x$  is the attenuator thickness, and  $\mu$  is the linear attenuation coefficient. A valuable indicator of a material's characteristic as a radiation attenuator is the mass attenuation coefficient ( $\mu_m$ ) which could be calculated using the formula below [31]:

$$\mu_m = \frac{\mu}{\rho}, \tag{2}$$

where,  $w_i$  denotes the weight fraction of  $i^{\text{th}}$ -element in the studied materials. The  $\mu_L$  parameter is necessary to calculate the values needed to reduce the primary irradiation to half or a tenth of its original value. The following equations can be used to calculate the attenuator thickness, known as the half-value layer (HVL), to reduce the strength of the photon by half its original value [3, 4]:

$$\text{HVL} = \frac{\ln 2}{\mu}, \tag{3}$$

The average distance gets through by a photon before it interacts with the attenuator substance is known as the mean free path ( $\lambda$ ) using Eq. 4 [22–26]:

$$\lambda = \frac{1}{\mu}, \tag{4}$$

The effective atomic number ( $Z_{\text{eff}}$ ) is also a significant parameter for the materials under investigation as an attenuator and the effective electron density  $N_{\text{eff}}$ . To get the  $Z_{\text{eff}}$  for a material, the atomic cross section ( $\text{ACS}(\sigma_a)$ ) and electronic cross section ( $\text{ECS}(\sigma_e)$ ) are initially determined. The following equations can be used to determine these parameters as follows [22, 23, 32, 33]:

$$\text{ACS} = \sigma_a = \sigma_m \frac{1}{\sum_i n_i} = (\mu/\rho)_{\text{target}} / N_A \sum_i \frac{w_i}{A_i}, \tag{5}$$

where,  $w_i$  is the fractional weight of the target individual element,  $A_i$  is the atomic weight,  $\sigma_m$  is the molecular cross-section, and  $N_A$  is Avogadro's constant. Additionally, the following formula can be used to determine ECS as follows [32–35]:

$$\text{ECS} = \sigma_e = \frac{1}{N_A} \sum_i \left( \frac{\mu}{\rho} \right)_i \frac{f_i A_i}{z_i}, \tag{6}$$

The target individual element's fractional abundance is represented by  $f_i$  and  $Z_i$  is the atomic number. So,  $Z_{\text{eff}}$  could be calculated as follows [36]:

**Table 2** Mass attenuation coefficient of the prepared ZnO–Al<sub>2</sub>O<sub>3</sub>–B<sub>2</sub>O<sub>3</sub>–SiO<sub>2</sub>–Gd<sub>2</sub>O<sub>3</sub> glasses via FLUKA and XCOM at different photon energies

Energy (MeV)	ZABSG1			ZABSG2			ZABSG3		
	XCOM	FLUKA	Dev.%	XCOM	FLUKA	Dev.%	XCOM	FLUKA	Dev.%
0.015	43.36539	43.23413	0.303	44.13146	43.91802	0.484	44.87034	44.60053	0.601
0.02	19.85432	19.89412	0.200	20.22264	20.25274	0.149	20.57789	20.60874	0.150
0.03	6.47799	6.49688	0.292	6.61096	6.62821	0.261	6.73921	6.75224	0.193
0.04	2.93109	2.93563	0.155	2.99591	2.99929	0.113	3.05843	3.06118	0.090
0.05	1.60931	1.60693	0.148	1.64653	1.64222	0.262	1.68243	1.67590	0.388
0.06	1.00811	1.00592	0.217	1.22044	1.21891	0.126	1.42523	1.42367	0.110
0.08	0.51562	0.51666	0.203	0.61578	0.61662	0.135	0.71239	0.71370	0.184
0.1	0.33284	0.33337	0.159	0.38791	0.38854	0.164	0.44102	0.44146	0.100
0.15	0.18664	0.18676	0.065	0.20480	0.20486	0.029	0.22231	0.22243	0.051
0.2	0.14303	0.14380	0.536	0.15121	0.15191	0.460	0.15910	0.15972	0.392
0.3	0.11029	0.11079	0.453	0.11291	0.11338	0.416	0.11544	0.11578	0.295
0.4	0.09514	0.09534	0.209	0.09629	0.09644	0.155	0.09739	0.09751	0.123
0.5	0.08549	0.08574	0.292	0.08608	0.08625	0.201	0.08664	0.08681	0.204
0.6	0.07843	0.07830	0.154	0.07875	0.07861	0.175	0.07905	0.07891	0.178
0.8	0.06834	0.06846	0.177	0.06844	0.06853	0.130	0.06853	0.06864	0.157
1.0	0.06121	0.06098	0.374	0.06123	0.06098	0.406	0.06124	0.06098	0.420
1.5	0.04982	0.04952	0.592	0.04978	0.04949	0.597	0.04975	0.04944	0.617
2	0.04326	0.04301	0.588	0.04325	0.04299	0.604	0.04324	0.04297	0.615
3	0.03607	0.03574	0.902	0.03612	0.03579	0.909	0.03617	0.03584	0.914
4	0.03231	0.03201	0.934	0.03242	0.03212	0.926	0.03252	0.03222	0.921
5	0.03010	0.02985	0.850	0.03026	0.03000	0.852	0.03041	0.03015	0.842
6	0.02873	0.02852	0.708	0.02892	0.02872	0.705	0.02911	0.02891	0.696
8	0.02728	0.02714	0.510	0.02754	0.02740	0.497	0.02779	0.02765	0.492
10	0.02668	0.02656	0.447	0.02700	0.02688	0.440	0.02730	0.02718	0.433
15	0.02652	0.02643	0.355	0.02694	0.02684	0.350	0.02734	0.02725	0.332

Energy (MeV)	ZABSG4			ZABSG5		
	XCOM	FLUKA	Dev.%	XCOM	FLUKA	Dev.%
0.015	46.27212	45.91177	0.779	47.58099	47.18871	0.824
0.02	21.25186	21.28863	0.173	21.88115	21.90798	0.123
0.03	6.98253	6.99448	0.171	7.20972	7.22107	0.157
0.04	3.17704	3.17725	0.007	3.28779	3.28609	0.051
0.05	1.75054	1.73895	0.662	1.81413	1.79805	0.887
0.06	1.81376	1.81242	0.074	2.17654	2.17526	0.059
0.08	0.89568	0.89736	0.188	1.06682	1.06888	0.193
0.1	0.54178	0.54192	0.026	0.63586	0.63586	0.000
0.15	0.25554	0.25565	0.041	0.28657	0.28652	0.015
0.2	0.17407	0.17472	0.373	0.18804	0.18853	0.262
0.3	0.12023	0.12052	0.240	0.12471	0.12495	0.194
0.4	0.09949	0.09956	0.070	0.10144	0.10148	0.034
0.5	0.08770	0.08782	0.130	0.08870	0.08876	0.074
0.6	0.07964	0.07947	0.215	0.08019	0.07999	0.243
0.8	0.06871	0.06879	0.115	0.06887	0.06892	0.071
1.0	0.06126	0.06099	0.447	0.06128	0.06099	0.483
1.5	0.04969	0.04931	0.767	0.04963	0.04924	0.772
2	0.04321	0.04294	0.637	0.04319	0.04291	0.642
3	0.03626	0.03593	0.911	0.03635	0.03600	0.954
4	0.03272	0.03243	0.888	0.03290	0.03261	0.879

**Table 2** (continued)

Energy (MeV)	ZABSG4			ZABSG5		
	XCOM	FLUKA	Dev. %	XCOM	FLUKA	Dev. %
5	0.03069	0.03043	0.845	0.03096	0.03070	0.834
6	0.02947	0.02926	0.687	0.02980	0.02959	0.686
8	0.02827	0.02813	0.480	0.02871	0.02857	0.471
10	0.02787	0.02775	0.432	0.02841	0.02829	0.416
15	0.02811	0.02801	0.335	0.02882	0.02873	0.334

$$Z_{eff} = \frac{\sigma_a}{\sigma_e}, \tag{7}$$

Build-up factors (*BUFs*) are important metrics demonstrating photon dispersion in radiation attenuation applications. We have to measure two parameters, *R* ratio, and *Z<sub>eq</sub>*, to determine the (*BUFs*) [32–36]:

$$R = \frac{(\mu/\rho)_{Com}}{(\mu/\rho)_{Total}}, \tag{8}$$

where,  $(\mu/\rho)_{Com}$  is the Compton  $\mu_m$  and  $(\mu/\rho)_{Total}$  is the total  $\mu_m$  for the target or attenuator. When materials absorb photons, a virtual atomic number (*Z<sub>eff</sub>*) describes the complex substance; this value is known as *Z<sub>eq</sub>* when photons are scattered. The following formula can be used to compute the build-up factors' second key parameter (*Z<sub>eq</sub>*) [32–36]:

$$Z_{eq} = \frac{Z_1(\log R_2 - \log R) + Z_2(\log R - \log R_1)}{\log R_2 - \log R_1}, \tag{9}$$

The two types of buildup factors (*BUFs*) are Energy Absorption Buildup Factor (*EABF*) and Exposure Buildup Factor (*EBF*). By understanding the (*R*) and (*Z<sub>eq</sub>*) parameters, one can calculate the geometric progression's (G-P) fitting parameters to compute the (*BUFs*) [36]:

$$P = \frac{P_1(\log Z_2 - \log Z_{eq}) + P_2(\log Z_{eq} - \log Z_1)}{\log Z_2 - \log Z_1}, \tag{10}$$

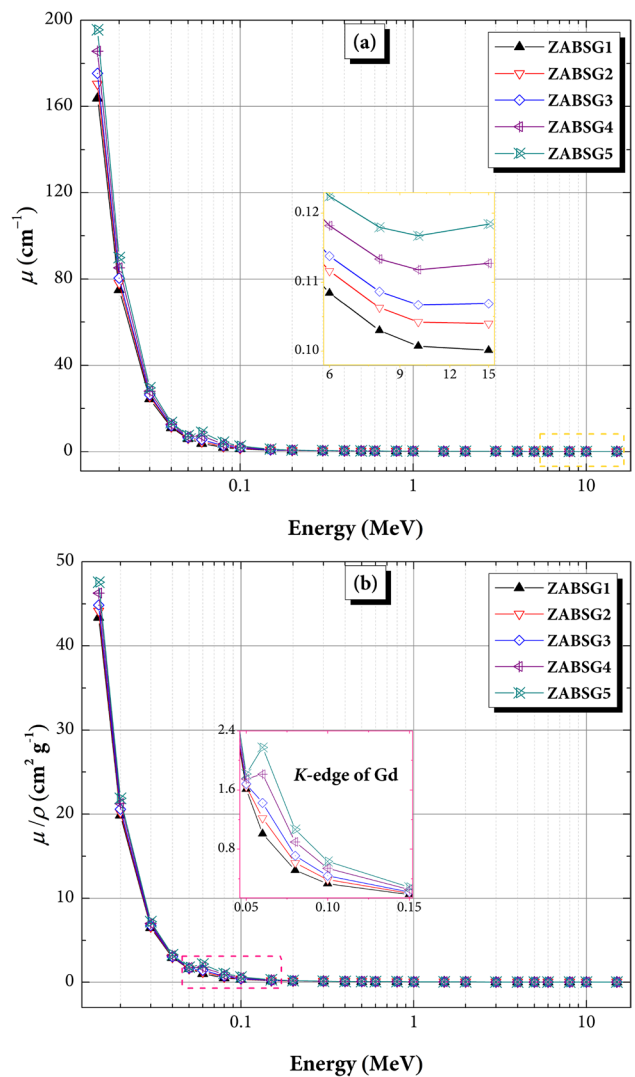
*Z<sub>1</sub>* and *Z<sub>2</sub>*'s respective atomic numbers are represented by the GP fitting parameters *P<sub>1</sub>* and *P<sub>2</sub>*, respectively. Then, employing GP-fitting to the following equations, *BUFs* were investigated:

$$B(E, X) = 1 + \frac{b-1}{K-1}(K^x - 1) \text{ for } K \neq 1, B(E, X) = 1 + (b-1)x \text{ for } K = 1, \tag{11}$$

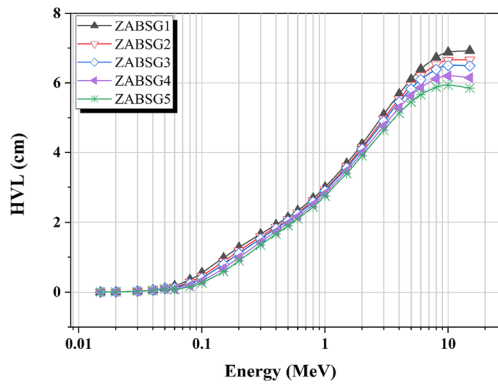
and,

$$K(E, X) = cx^a + d \frac{\tanh(\frac{x}{Xk} - 2) - \tanh(-2)}{1 - \tanh(-2)} \text{ for } x \leq 40, \tag{12}$$

where (*a*, *b*, *c*, *xk*, and *d*) are G-P fitting parameters, *E* is the energy photon, *x* is the penetration depth in  $\lambda$ , and *K(E, X)* is the dose-multiplicative factor.



**Fig. 2** (a) Linear attenuation coefficient ( $\mu$ ) and (b) mass attenuation coefficient ( $\mu/\rho$ ) of the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%) with different photon energies



**Fig. 3** Half value layer (HVL) with respect to the concentration of  $\text{MoO}_3$  and a function of photon energy in the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%)

## 2.5 Neutron Rays Attenuation

The  $FNRCs$  ( $\Sigma_R$ ) is a significant measure to evaluate the planned material systems' capacity for neutron attenuation.

Knowing that the sample density represents by  $(\rho)_s$ ,  $w_i$  is the  $i^{\text{th}}$ -ingredient weight fraction,  $W_i$  stands for partial density and  $\left(\frac{\Sigma_R}{\rho}\right)_i$  is a representation of the  $i^{\text{th}}$  component's mass removal cross-section, so any absorber's  $\Sigma_R$  can be calculated as follow [2, 5, 14, 15]:

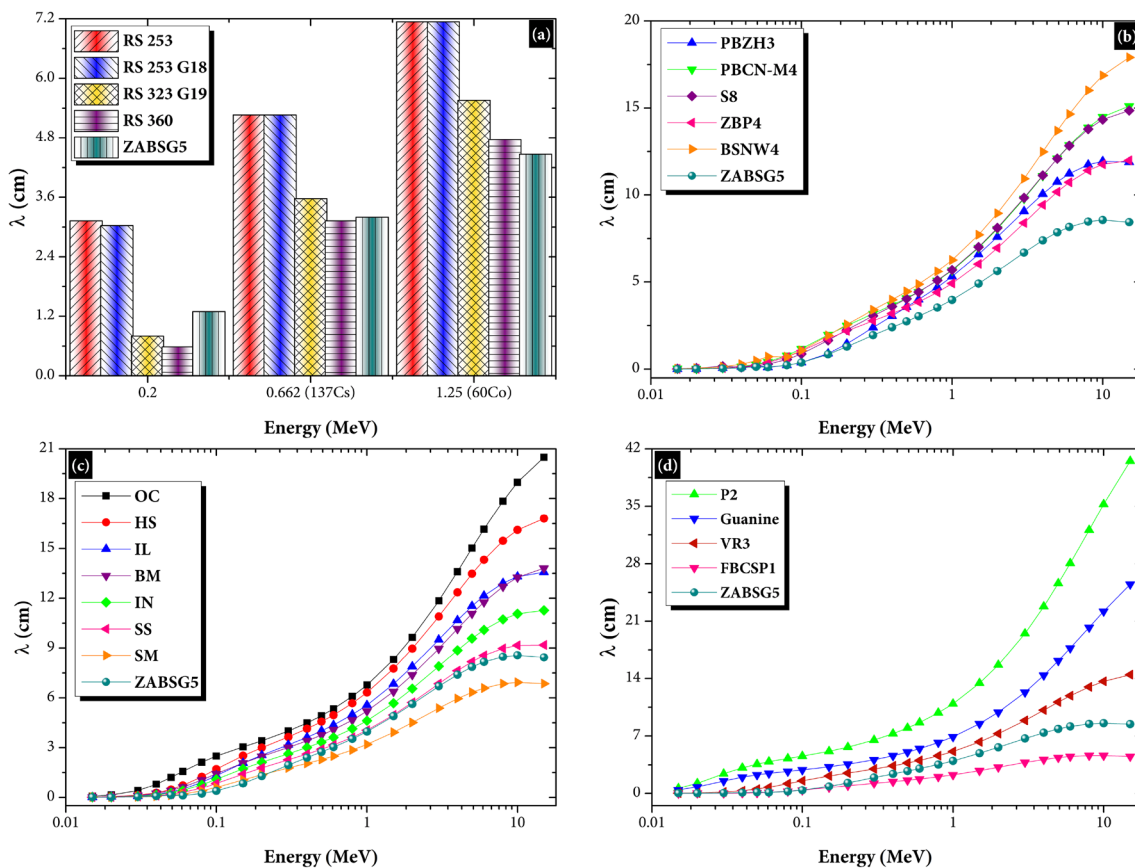
$$\frac{\Sigma_R}{\rho} = \sum_i W_i \left(\frac{\Sigma_R}{\rho}\right)_i, \quad (13)$$

where,

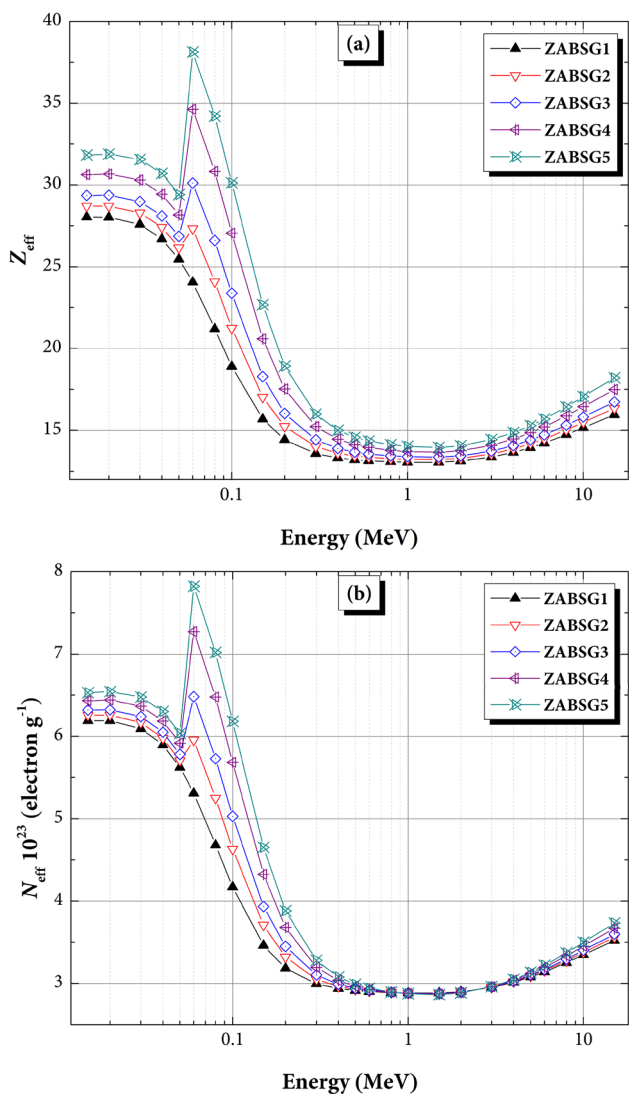
$$W_i = \sum_i w_i(\rho)_s. \quad (14)$$

## 3 Results and Discussion

The  $\mu/\rho$  values for the  $x\text{Gd}_2\text{O}_3 - (60-x)\text{ZnO}-15\text{B}_2\text{O}_3-5\text{Al}_2\text{O}_3-20\text{SiO}_2$  glass system ( $x=0-3$  mol %) were assessed using FLUKA and XCOM software to assess the radiation attenuation characteristics. The relative difference (Div. %) between the results obtained from FLUKA and XCOM are listed in the Table 2. The results showed good agreement



**Fig. 4** Comparison of mean free path ( $\lambda$ ) parameter of the prepared ZABSG5 sample with those in (a) commercial SCHOTT's radiation shielding glasses [1], (b) some glass systems [2–6], (c) standard shielding concretes [7], and (d) polymer-composite materials [8–11]



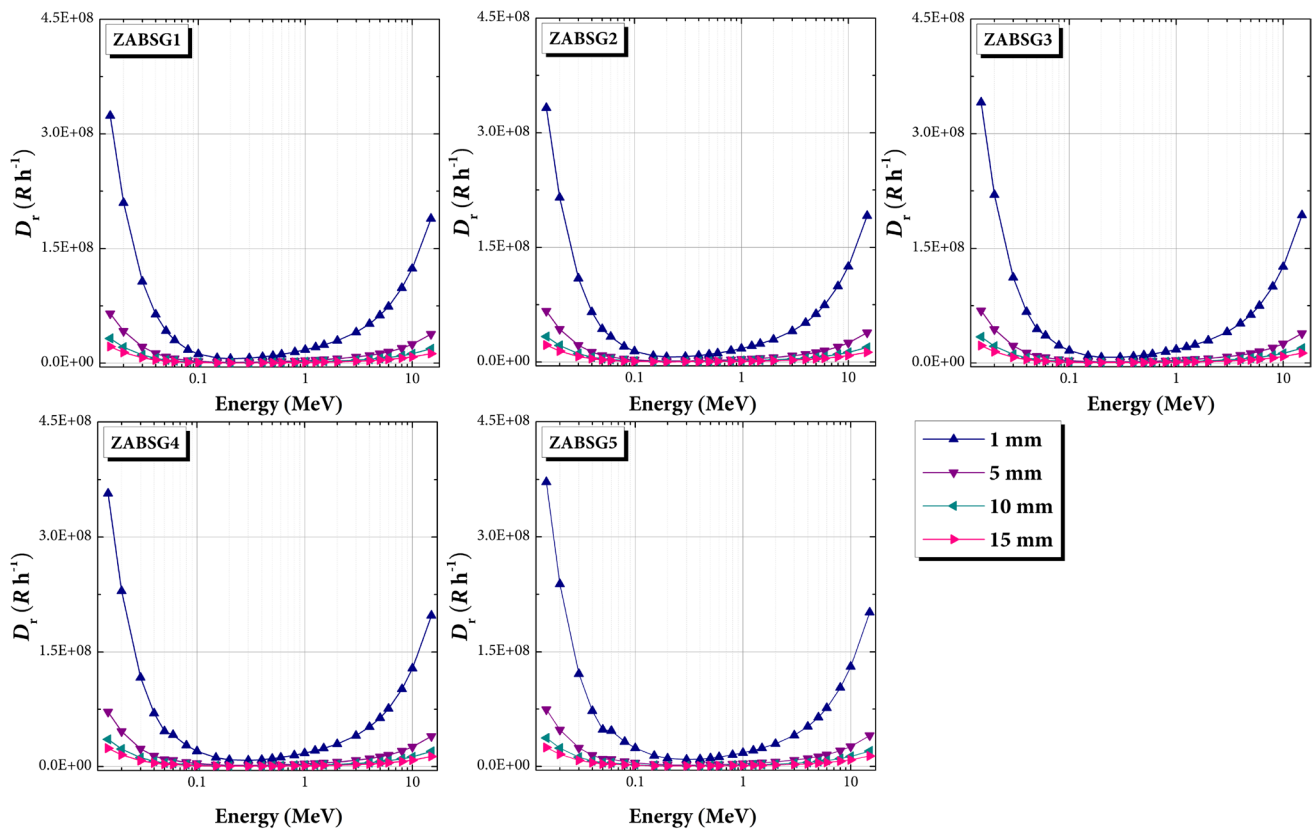
**Fig. 5** **a** Effective atomic number and **(b)** effective electron density of the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%), with different photon energies

with a maximum relative difference of 0.9540%. On the other hand, Fig. 2(b) represents that there is a strong reduction in the  $\mu/\rho$  values for all prepared samples due to Photo electric effect interaction (PEEI) which has cross-section changes with  $E_\gamma^{-3.5}$ . Therefore, the interaction cross-section reduced strongly with the enrichment of photon values which is associated with a similar reduction in the photon-electron interactions and  $\mu_L$  values. The enrichment of the applied  $E_\gamma$  values between 0.015–0.2 MeV causes a strong exponential decrease in the values of  $\mu/\rho$  ( $\text{cm}^2.\text{g}^{-1}$ ) trends from 43.2341 to 0.1438 for ZABSG1, from 43.9180 to 0.1512 for ZABSG2, from 44.60053 to 0.15972 for ZABSG3, from 45.9117 to 0.1747 for ZABSG4 and from 47.1887 to 0.1885 for ZABSG5. Furthermore, the enrichment of  $E_\gamma$  values above 0.2 MeV causes an exponential decrease

in the  $\mu/\rho$  values in the  $E_\gamma$  interval of 0.2 to 2 MeV. The exponential reduction is attributed to the Compton scattering interaction (Com) with cross-section changes with  $E_\gamma^{-1}$ . Photons with more kinetic energy have a lower probability of interacting with atoms in the substance [15]. Therefore, the probability of absorption decreases associated with the increased probability of photons scattering with increasing energy. The enrichment in  $E_\gamma$  values was associated with a smooth decrease in the cross-section with decreases in the number of photon-electron interactions followed by a smooth reduction in the  $\mu/\rho$  values. There is a reduction in the  $\mu/\rho$  values (in the unit of  $\text{cm}^2.\text{g}^{-1}$ ) from 0.1107 to 0.0430 for ZABSG1, from 0.1133 to 0.0429 for ZABSG2, from 0.1157 to 0.0429 for ZABSG3, from 0.1205 to 0.0429 for ZABSG4 and from 0.1249 to 0.0429 for ZABSG5 with raising the  $E_\gamma$  values between 0.3 MeV and 2 MeV, respectively. Also, there is a little reduction due to the pair production interaction (PP) with cross-section changes with  $E_\gamma^2$ . The  $\mu/\rho$  values, in this region vary from 0.0357 to 0.0264 for ZABSG1, from 0.0357 to 0.0268 for ZABSG2, from 0.0358 to 0.0272 for ZABSG3, from 0.0359 to 0.0280 for ZABSG4 and from 0.0360 to 0.0287 for ZABSG5 with raising the  $E_\gamma$  values between 3–15 MeV. The  $\mu/\rho$  values for ZABSG5 are generally the highest among the other prepared materials due to the high concentration of gadolinium element (11.1229%). Figure 2(a) shows The linear attenuation coefficient ( $\mu$ ,  $\text{cm}^{-1}$ ) of the prepared samples, which take the same behavior as the  $\mu/\rho$  for the investigated samples. The  $\mu$  values of ZABSG5 have higher than ZABSG1, ZABSG2, ZABSG3, and ZABSG4 at the same energy energies.

The half-value layer (HVL, cm) values for the synthetic  $\text{Gd}_2\text{O}_3$  glass samples are displayed in Fig. 3 for various energies between 0.015–15 MeV. The HVL values for the studied samples increase as the energy of the radiation increases, which is expected. It's the thickness of the investigated samples that can shield half of the incident gamma photons. At lower and mid energy ranges (0.015 – 0.2 MeV), there are significant differences in the HVL values among the studied materials, but these differences decrease at the higher energies (0.3 – 2 MeV) and (3 – 15 MeV). The HVL values ranged from 0.00423 To 6.9236 cm for ZABSG1, from 0.0040 to 6.6660 for ZABSG2, from 0.0039 to 6.4853 cm for ZABSG3, from 0.0037 to 6.1483 for ZABSG4 and from 0.0035 to 5.8528 cm for ZABSG5 in the energy of 0.015 – 15 MeV. It suggests that ZABSG5 has the lowest values of HVL and may be better suited for radiation shielding.

Mean free path ( $\lambda$ ) is a standard measure of radiation protection effectiveness. It also demonstrates whether or not the shielding material is of sufficient thickness. Since radiation is dampened by traveling through a thinner region, the lower values for either parameter often result in higher radiation shielding performance for a given photon energy. Typically, the  $\lambda$  all go up and down in tandem. Figure 4(a-d)

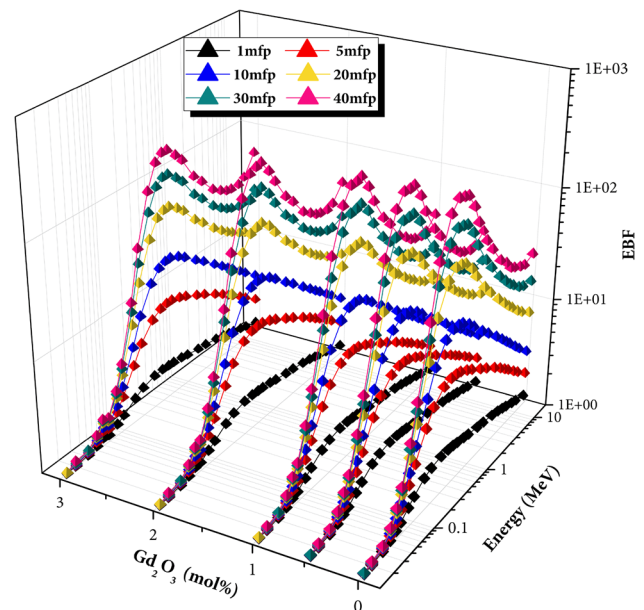


**Fig. 6** Gamma dose rate ( $D_r$ ) at different photon energy levels for the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%)

compares the best-prepared sample ZABSG5 with those of different standard materials (e.g., SCHOTT's radiation shielding glasses (<https://www.schott.com/en-gb>), (b) some glass systems [37–41], (c) standard shielding concretes [10], and (d) polymer-composite materials [42–44]. Figure 4(a, b) show that the investigated ZABSG5 sample has the lowest values of  $\lambda$  with those of commercial SCHOTT's glass (RS-253, RS-253-G18, RS-323-G19, RS-360, and other glass samples) at photon energy 1.25 MeV. On the contrary, the ZABSG5 sample has a value higher than RS-323-G19 and RS-360 glass samples at photon energy 0.2 MeV.

Also, the investigated ZABSG5 glass sample has the lowest values of  $\lambda$  with all the compared standard concrete, Ordinary concrete (OC), Hematite-serpentine concrete (H-S), Ilmenite limonite concrete (IL), Basalt-magnetite concrete (B-M), Ilmenite concrete (IN) and Steel-scrap concrete (S-S) except the Steel magnetite (S-M) as shown in Fig. 4(c). Figure 4(d) shows that the prepared ZABSG5 sample has the lowest values of  $\lambda$  with those of polymer-composite (P2, Guanine, and VR3) except for the FBCSP1 sample.

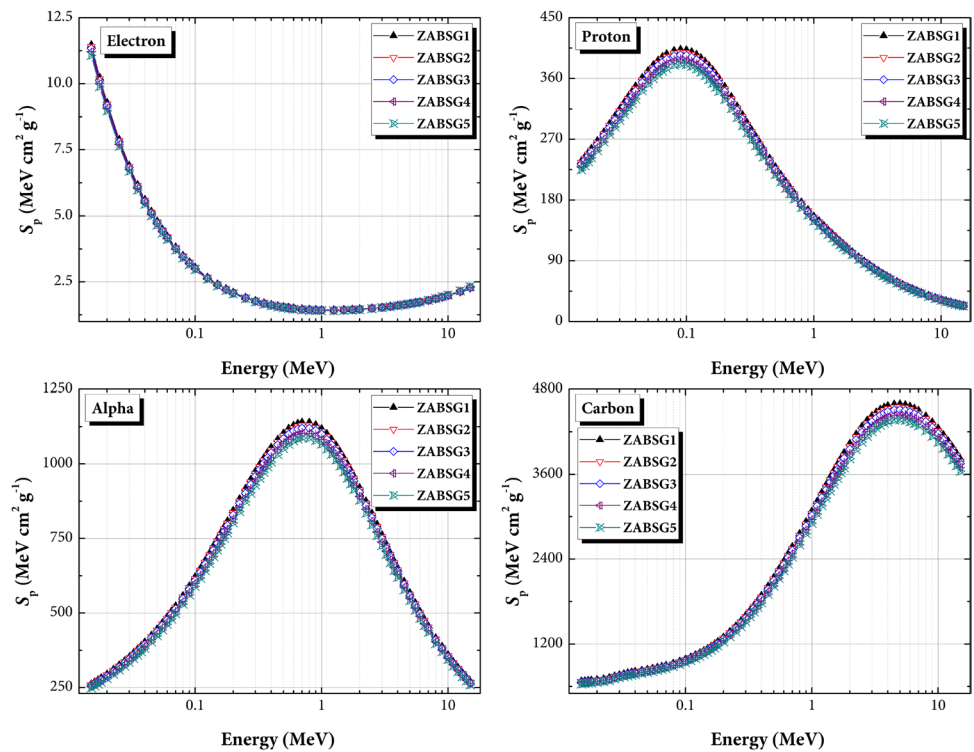
Figure 5(a, b) presents the investigated glass samples' atomic effective number ( $Z_{\text{eff}}$ ) values and effective electron



**Fig. 7** Exposure buildup factor (EBF) with the photon energy for the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%)



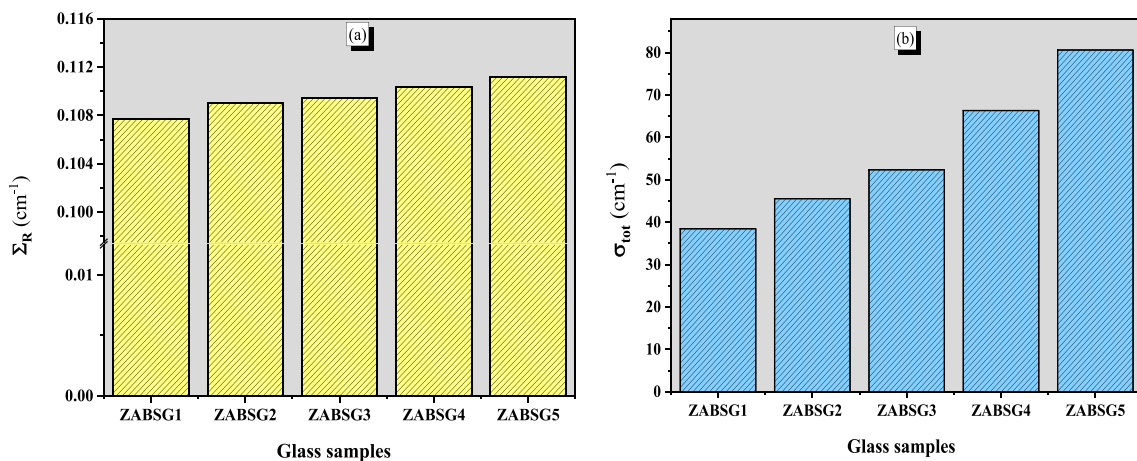
**Fig. 8** Total mass stopping power with respect to a function of kinetic energy in the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%) presented for electron, proton & alpha particles, and carbon ion



density ( $N_{\text{eff}}$ ) vs. photon energy in the energy range of 0.015 – 15 MeV. In general, the higher  $Z_{\text{eff}}$  and  $N_{\text{eff}}$  values indicate a greater ability of a material to interact with radiation, particularly through the Com and PEEI regions [45]. Therefore, materials with higher  $Z_{\text{eff}}$  values may be more effective in shielding against high-energy radiation. It indicates that the effectiveness of the materials in attenuation against radiation is dependent on the energy of the radiation and that a different material may be more effective at higher or lower MeV energy. At lower MeV values between 0.015–0.2, the ZABSG5 has the highest

$Z_{\text{eff}}$  and  $N_{\text{eff}}$  values among the four materials. As the MeV value increases, the  $Z_{\text{eff}}$  values for the studied materials decrease, but ZABSG5 still has the highest  $Z_{\text{eff}}$  and  $N_{\text{eff}}$  values. Also, the ZABSG1 compound has the lowest  $Z_{\text{eff}}$  values among the four materials.

Figure 6 displays the  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glass systems' computed gamma dose rate ( $D, \text{R.h}^{-1}$ ) spectra outside glasses with thicknesses ranging from 1–15 mm.  $D$ 's energy-dependent fluctuations at the various thicknesses under consideration show that gamma dose rate constant ( $C$ ) has a significant impact on the variation.  $D$  also



**Fig. 9** (a) Removal ( $\Sigma_R$ ), and (b) total scattering ( $\sigma_{\text{tot}}$ ) cross sections of the prepared  $(60-x)\text{ZnO}-5\text{Al}_2\text{O}_3-15\text{B}_2\text{O}_3-20\text{SiO}_2-x\text{Gd}_2\text{O}_3$  glasses (where  $x=0, 0.5, 1, 2,$  and  $3$  mol%) for fast and thermal neutrons

**Table 3** Fast neutron removal cross-section of elements contained in the prepared ZnO–Al<sub>2</sub>O<sub>3</sub>–B<sub>2</sub>O<sub>3</sub>–SiO<sub>2</sub>–Gd<sub>2</sub>O<sub>3</sub> glasses

Glass code	Element	$\Sigma_R/\rho$ (cm <sup>2</sup> /g)	Fraction by weight, $w_i$	Partial density (g/cm <sup>3</sup> ), $W_i$	$\Sigma_R$ (cm <sup>-1</sup> )
ZABSG1	B	0.05746705	0.0424562	0.16027231	0.00921038
	O	0.04052824	0.3351028	1.26501322	0.05126876
	Al	0.02934486	0.0353200	0.13333313	0.00391264
	Si	0.02814153	0.0735303	0.27757704	0.00781144
	Zn	0.01829517	0.5135905	1.93880429	0.03547076
	Gd	0.01192390	0.0424562	0.00000000	0.00000000
					$\Sigma_{R=}$
Total $\Sigma_R$ for glass 'ZABSG1' = <b>0.10767</b> cm <sup>-1</sup>					
ZABSG2	B	0.05746705	0.0416892	0.16092029	0.00924761
	O	0.04052824	0.3311052	1.27806593	0.05179776
	Al	0.02934486	0.0346819	0.13387220	0.00392846
	Si	0.02814153	0.0722019	0.27869928	0.00784302
	Zn	0.01829517	0.5001090	1.93042080	0.03531738
	Gd	0.01192390	0.0202128	0.07802151	0.00093032
					$\Sigma_{R=}$
Total $\Sigma_R$ for glass 'ZABSG2' = <b>0.10906</b> cm <sup>-1</sup>					
ZABSG3	B	0.05746705	0.0409494	0.16007109	0.00919881
	O	0.04052824	0.3272494	1.27921780	0.05184445
	Al	0.02934486	0.0340664	0.13316573	0.00390773
	Si	0.02814153	0.0709206	0.27722854	0.00780163
	Zn	0.01829517	0.4871060	1.90409728	0.03483579
	Gd	0.01192390	0.0397083	0.15521955	0.00185082
					$\Sigma_{R=}$
Total $\Sigma_R$ for glass 'ZABSG3' = <b>0.10944</b> cm <sup>-1</sup>					
ZABSG4	B	0.05746705	0.0395458	0.15861820	0.00911532
	O	0.04052824	0.3199343	1.28325638	0.05200812
	Al	0.02934486	0.0328988	0.13195705	0.00387226
	Si	0.02814153	0.0684897	0.27471226	0.00773082
	Zn	0.01829517	0.4624370	1.85483473	0.03393452
	Gd	0.01192390	0.0766944	0.30762139	0.00366805
					$\Sigma_{R=}$
Total $\Sigma_R$ for glass 'ZABSG4' = <b>0.11033</b> cm <sup>-1</sup>					
ZABSG5	B	0.05746705	0.0382353	0.15710866	0.00902857
	O	0.04052824	0.3131040	1.28654442	0.05214138
	Al	0.02934486	0.0318085	0.13070124	0.00383541
	Si	0.02814153	0.0662200	0.27209788	0.00765725
	Zn	0.01829517	0.4394030	1.80550707	0.03303206
	Gd	0.01192390	0.1112292	0.45704073	0.00544971
					$\Sigma_{R=}$
Total $\Sigma_R$ for glass 'ZABSG5' = <b>0.11114</b> cm <sup>-1</sup>					

varies negatively with glass thickness at all energy levels and types of glass. When the glass thickness and the photon energy remain constant, D varies negatively with  $Z_{\text{eff}}$ . The EBF and EABF values of the examined compounds fluctuate with photon energy at 1,–40 Å, as shown in Fig. 7. The photon energy, sample composition, and penetration depth influence the maximum values of the BUF. Multiple scatterings

happen at greater penetration depths. The highest BUFs values were found at a penetration depth of 40 MFPs, and the lowest were found at a penetration depth of 1. The values of the BUFs rise with photon energy to their maximum value before falling as photon energy rises. The PEEI dominates interactions in the lower energy area; hence many photons have been absorbed, resulting in the lowest BUFs. Due to the prevailing

Com, which scatters the photon's energy but cannot destroy it, the region of intermediate photon energy has the highest BUFs values. The photons were once more absorbed in the higher energy region where pair formation is the dominant interaction. Figure 8(a-d) The electron, proton, alpha, and carbon mass stopping power ( $E/P/A/CS_p$ ) and calculated range values of  $(60-x)ZnO-5Al_2O_3-15B_2O_3-20SiO_2-xGd_2O_3$  glasses were calculated by using FLUKA code. The  $MS_p$  results are important as they indicate the relation of the electron, proton, alpha, and carbon energy loss when they pass through the glass system. As shown in Fig. 8(a), ESP values drop as kinetic energy increases.

Moreover, the  $ES_p$  values drop with increasing  $Gd_2O_3$  concentration, with the lowest  $AS_p$  values found in the ZABSG5 glass system. As presented in Fig. 8(b), maximum  $PS_p$  values are reached at a kinetic energy of 0.080 MeV and then drop when the kinetic energy increases. Furthermore, The  $PS_p$  values decrease with the increase in  $Gd_2O_3$  concentration. The ZABSG5 glass system has the lowest PMSP values in kinetic energy. Figure 8(c), when the kinetic energy increases to a maximum of 0.8 MeV, AMSP values increase, and in case of increasing the kinetic energy values, the AMSP values decrease.

Furthermore, The  $AS_p$  values decrease with the increase in  $Gd_2O_3$  concentration. The prepared ZABSG5 glass system has the lowest  $AS_p$  values in the kinetic energy range of 0.015–15 MeV. Figure 8(d) shows that the  $CS_p$  values increase to a kinetic energy of about 5 MeV, then fall when the kinetic energy is increased further. In addition, in the kinetic energy range of 0.015–15 MeV, the  $CS_p$  values are lowest for the ZABSG5 glass system, which decreases as  $Gd_2O_3$  concentration increases. ZABSG5 glass has the greatest density ( $4.109 \text{ g.cm}^{-3}$ ) and the highest content of high atomic number ( $Gd=64$ ), which may account for these characteristics.

The fast neutrons removal cross section shows the probability that neutrons will pass through a substance without reacting ( $\Sigma_R, \text{cm}^{-1}$ ). As seen in Fig. 9(a, b), The  $\Sigma_R$  of the studied samples have values of 0.1076, 0.1090, 0.1094, 0.1103, and 0.1111  $\text{cm}^{-1}$  for ZABSG1, ZABSG2, ZABSG3, ZABSG4 and ZABSG5 respectively. ZABSG5 has the highest value of the  $\Sigma_R$ , and ZABSG1 has the lowest due to the concentration of gadolinium (Gd) element used in fuel assemblies as a neutron absorber. Also, the prepared samples' thermal neutrons cross section ( $\sigma_{tot}, \text{cm}^{-1}$ ) were calculated. The  $\sigma_{tot}$  of the studied glass samples have the same behavior. The  $\sigma_{tot}$  of the prepared glass samples have 38.4298, 45.5252, 52.3455, 66.3203, and 80.5895 for ZABSG1, ZABSG2, ZABSG3, ZABSG4 and ZABSG5 respectively. ZABSG5 has the highest value of the  $\sigma_{tot}$ , and ZABSG1 has the lowest, as listed in Table 3. Therefore, we may deduce that Gd element doping gives the studied compound a high potential across the neutrons shielding Table 4.

**Table 4** Coherent scattering cross-section ( $\sigma_{coh}$ ), incoherent scattering cross-section ( $\sigma_{inc}$ ), absorption cross-section ( $\sigma_{abs}$ ), and total cross-section ( $\sigma_{tot}$ ) of the prepared  $ZnO-Al_2O_3-B_2O_3-SiO_2-Gd_2O_3$  glasses for thermal neutrons attenuation

Sample code	Thermal neutron cross section ( $\text{cm}^{-1}$ )			
	$\sigma_{coh}$	$\sigma_{inc}$	$\sigma_{abs}$	$\sigma_{tot}$
ZABSG1	0.95991	0.08416	37.38575	38.42982
ZABSG2	0.98389	0.10507	44.43632	45.52528
ZABSG3	0.99874	0.12546	51.22135	52.34555
ZABSG4	1.02958	0.16721	65.12357	66.32036
ZABSG5	1.05953	0.20990	79.32015	80.58958

## 4 Conclusion

This research reported on the shielding qualities of  $xGd_2O_3 - (60-x)ZnO-15B_2O_3-5Al_2O_3-20SiO_2$  glass system ( $x=0-3 \text{ mol } \%$ ) for gamma photon starting from 15 keV up to 15 MeV. The results of the  $\mu_m$  obtained from FLUKA and XCOM show a good agreement with a maximum relative difference of 0.901%. Also, the  $\mu_m$  values, in the unit of  $\text{cm}^2.\text{g}^{-1}$ , for the investigated system were varied from 43.2341 to 0.0264 for ZABSG1, 43.918 to 0.02684 for ZABSG2, 44.6005 to 0.0272 for ZABSG3, 45.9117 to 0.0280 for ZABSG4 and 47.1887 to 0.0287 for ZABSG5 in the energy range of 0.015 – 15 MeV. Also, The FNRCs values were 0.1076, 0.1090, 0.1094, 0.1103, and 0.1111  $\text{cm}^{-1}$  for ZABSG1, ZABSG2, ZABSG3, ZABSG4, and ZABSG5 respectively. In addition, The TNRCs values were 38.4298, 45.5252, 52.3455, 66.3203, and 80.5895 for ZABSG1, ZABSG2, ZABSG3, ZABSG4, and ZABSG5 respectively. The glass sample ZABSG5 had the highest values of  $\mu_m$ ,  $\mu$ , and T/FNRCs and the lowest values of HVL and  $\lambda$  due to the high concentration of  $Gd_2O_3$  (11.122%) and density ( $4.109 \text{ g.cm}^{-3}$ ) and  $Z_{eff}$ . The results referred to the high radiation shielding potential for the present glassy materials as compared with previously studied shielding materials, which should aid in developing effective radiation shielding materials for use in the nuclear and medical industries.

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**Data Availability** The data that support the findings of this study are available from the corresponding author upon reasonable request.

**Code Availability** Not applicable.

## Declarations

**Ethics Approval** Not applicable.

**Consent to Participate** Not applicable.

**Consent for Publication** Not applicable.

**Competing Interests** The authors declare no competing interests.

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