# A simple acid digestion using HCI–HNO<sub>3</sub>–NH<sub>4</sub>HF<sub>2</sub> for rapid SF-ICP-MS determination of <sup>237</sup>Np and Pu isotopes in steel and concrete samples

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Received: 29 April 2021 / Accepted: 4 June 2021 / Published online: 21 June 2021 © Akadémiai Kiadó, Budapest, Hungary 2021

#### Abstract

A rapid method was developed to analyze <sup>237</sup>Np and Pu isotopes in steel and concrete samples using HCl-HNO<sub>3</sub>-NH<sub>4</sub>HF<sub>2</sub> digestion, CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation, AG MP-1 M resin column separation and SF-ICP-MS measurement. Compared to acid digestion using HCl-HNO<sub>3</sub>-HF, the whole procedure not only avoids the use of toxic HF, but can be completed within 1 day. The decontamination factor of U was estimated to be  $1.1 \times 10^6$ . The limits of detection of <sup>237</sup>Np, <sup>239</sup>Pu, and <sup>240</sup>Pu were 0.25, 0.16, and 0.14 fg  $g^{-1}$ , respectively, allowing method users to sufficiently detect low level Pu and Np in steel and concrete samples.

Keywords Steel · Concrete · Plutonium · Neptunium · SF-ICP-MS

# Introduction

Nuclear energy as a clean and economical electrical energy is favored by many countries. At present, there are more than 400 nuclear power reactors in operation, which provide about 11% of the electrical power generated worldwide [1, 2]. However, emergencies and accidents that come with the operation of nuclear power plants need to be faced, such as the Fukushima Daiichi Nuclear Power Plant accident that occurred in March 2011. In addition, the number of first-generation nuclear power plants that need to be decommissioned has also been increasing gradually, since the typical design life of a nuclear power reactor was 30-50 years [2, 3]. During the decommission and decontamination, the resulting various types of radioactive wastes need to be safely managed and disposed of; these include steel and concrete materials used in buildings, shielding and the pressure vessel of the nuclear reactor, etc. [4, 5]. In order to establish appropriate disposal strategies, the inventory and distribution of

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the target radionuclides (including fission products and actinides) should be measured. Actinide radionuclides, especially Np and Pu are the key radionuclides of concern for the characterization of nuclear decommissioning samples due to their long half-lives (e.g., <sup>237</sup>Np ( $t_{1/2} = 2.14 \times 10^6$  y), <sup>239</sup>Pu ( $t_{1/2} = 2.41 \times 10^4$  y), and <sup>240</sup>Pu ( $t_{1/2} = 6.56 \times 10^3$  y)), radioactivity, and toxicity. Weinreich et al. [5] have reported that the concentration of U is at the  $\mu g g^{-1}$  level in bio-shielding concrete materials; and <sup>237</sup>Np and <sup>239</sup>Pu will be generated via the following reactions [5, 6]:

<sup>235</sup>U(n, 
$$\gamma$$
)<sup>236</sup>U(n,  $\gamma$ )<sup>237</sup>U $\xrightarrow{6.75d,\beta^{-}}$ <sup>237</sup>Np  
<sup>238</sup>U(n, 2n)<sup>237</sup>U $\xrightarrow{\beta^{-}}$ <sup>237</sup>Np  
<sup>238</sup>U(n,  $\gamma$ )<sup>239</sup>U $\xrightarrow{235m,\beta^{-}}$ <sup>239</sup>Np $\xrightarrow{2.355d,\beta^{-}}$ <sup>237</sup>Pu

Therefore, rapid and simple methods to analyze Np and Pu in steel and concrete samples are highly desired for preparing responses to a nuclear accident during reactor operation and decommission based on reliable risk assessment.

Steel and concrete can be divided into materials that have induced internal radioactivity and those that have only surface radioactivity depending on the source of the radionuclides [4]. If the buildings of concern are far away from the nuclear reactor, the radioactive substances will be only deposited loosely onto the bare surface of the building materials after a nuclear accident. By contrast, for the steel and concrete materials in the pressure vessel and shielding of



the nuclear reactor, they are close to the reactor core and have had long-term exposure to the neutron flux of the reactor fuels, resulting in the radionuclides being present inside and on the surfaces of the steel and concrete. Therefore, total digestion of analysis samples is critical to release radionuclides completely from the steel and concrete materials. Aqua regia and dilute  $H_2SO_4$  (3–6 M) have been reported to be effective in complete digestion of steel samples [4, 7, 8]. However, the presence of sulfate ions will lead to CaSO<sub>4</sub> precipitate formation during the process of CaF<sub>2</sub>/ LaF<sub>3</sub> co-precipitation, which cannot be dissolved rapidly in the following HNO<sub>3</sub> solution prior to resin column chromatography. Digestion of concrete only using aqua regia cannot release Pu completely due to potential hot particles or refractory matrixes [9, 10]. Although many methods for the analysis of Np and Pu isotopes in soil or sediment have been reported [6, 11-16], they are not feasibly applicable to analyze Np and Pu in concrete, due to not only the potential hot particles or refractory matrixes in the concrete samples, but also the time-consuming procedures of these methods, which cannot meet the requirement for the rapid response in an emergency [17, 18]. To date, a few studies have reported methods to digest concrete samples for the analysis of Pu [5, 10, 17, 18], such as HNO<sub>3</sub>-H<sub>3</sub>BO<sub>3</sub>-HF digestion [5], HNO<sub>3</sub>-HClO<sub>4</sub>-HF digestion [17], and HCl-HNO<sub>3</sub>-HF digestion or NaOH fusion [10, 18]. However, it should be noted that the previous methods for the total digestion of concrete samples are tedious, time-consuming and hazardous as they have: 1) repeated acid digestions, 2) fusion with NaOH (high temperature (600 °C)), and 3) use of toxic HF. Therefore, a proper decomposition approach avoiding these disadvantages is highly required for the measurement of Np and Pu isotopes in steel and concrete samples. Ammonium hydrogen fluoride (NH4HF2) has been used to digest soil or rock samples at low temperature [19–21], since it partially decomposes into HF at temperatures between ~ 120 °C and 220 °C [19], and it also can directly decompose the sample by breaking the Si–O bond and Pu–O bond [20]. In addition, Yang et al. [13] found that  $NH_4HF_2$  could release  $F^-$  in  $H^+$ solutions. Therefore, NH<sub>4</sub>HF<sub>2</sub> is a potential replacement reagent for toxic HF to get complete digestion of concrete samples and then to form CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitate to remove matrix and interfering elements, especially U, for the final <sup>237</sup>Np and Pu isotopes analysis by sector field-inductively coupled plasma-mass spectrometry (SF-ICP-MS).

SF-ICP-MS was employed for the analysis of <sup>237</sup>Np and Pu isotopes in this study, based on its characteristics of high sensitivity and short measurement time [22]. It should be noted that high removal of U is required for accurate measurement <sup>237</sup>Np and Pu isotopes in steel and concrete samples by SF-ICP-MS, due to the tailing effect of the <sup>238</sup>U signal overlapping the <sup>237</sup>Np signal and the dominant polyatomic interferences of the <sup>238</sup>UH<sup>+</sup> and <sup>238</sup>UH<sub>2</sub><sup>+</sup> on <sup>239</sup>Pu and <sup>240</sup>Pu [12, 23, 24]. It was reported that about 70% of Fe and 60% of U was removed by CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation [23], and a high decontamination factor  $(1.6 \times 10^7)$  of U was achieved using CaF<sub>2</sub>/LaF<sub>2</sub> co-precipitation combined with AG MP-1 M resin column chromatography in analysis of <sup>237</sup>Np and Pu isotopes in soil and sediment samples [12]. Therefore, CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation combining with the AG MP-1 M resin column chromatography was selected here to separate and purify Np and Pu isotopes in steel and concrete samples. Furthermore, there is no proper yield tracer for <sup>237</sup>Np analysis by ICP-MS, since most Np isotopes have short half-lives [6]: for example,  $^{239}$ Np ( $t_{1/2} = 2.4$ d) and <sup>235</sup>Np ( $t_{1/2}$  = 396 d). Although <sup>236</sup>Np ( $t_{1/2}$  = 1.54 × 10<sup>5</sup> y) is an ideal tracer, it is not commercially available currently. <sup>242</sup>Pu with a long half-life  $(3.76 \times 10^5)$  has often been used as a yield tracer for the determination of Pu isotopes by ICP-MS. Tetravalent Np and Pu have been reported to have similar chemical behavior [17] and the first ionization potentials of Np and Pu are similar (6.26 eV and 6.03 eV, respectively) [25]. Meanwhile, <sup>242</sup>Pu as a yield tracer for the analysis of <sup>237</sup>Np has been proposed in previous studies [11–16]. Therefore, the non-isotopic  $^{242}$ Pu was also chosen to see if it could act as a yield tracer for the simultaneous measurement of <sup>237</sup>Np and Pu isotopes by SF-ICP-MS after HCl-HNO<sub>3</sub>-NH<sub>4</sub>HF<sub>2</sub> total digestion, CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation, and AG MP-1 M resin column separation.

In this work, a novel HCl–HNO<sub>3</sub>–NH<sub>4</sub>HF<sub>2</sub> total digestion method for steel and concrete samples was developed to analyze <sup>237</sup>Np and Pu isotopes simultaneously by SF-ICP-MS. Since no steel and concrete standard reference material is commercially available, one marine sediment was used (collected from the Sea of Japan in Ishikawa Prefecture), with known amounts of <sup>239</sup>Pu, <sup>240</sup>Pu, and <sup>237</sup>Np measured by the method of Huang et al. [12] and treated at high temperature (1000 °C) as standard reference material to assess the accuracy of the developed method.

## Experimental

## **Reagents and materials**

Analytical grade reagents including HNO<sub>3</sub> (60–61%), HCl (35–37%), HF (46–48%), HBr (47–49%), NH<sub>4</sub>HF<sub>2</sub>, H<sub>3</sub>BO<sub>3</sub>, Ca(NO<sub>3</sub>)<sub>2</sub>, La(NO<sub>3</sub>)<sub>3</sub>, TiCl<sub>3</sub> (20%), NaNO<sub>2</sub>, and ascorbic acid were purchased from Kanto Chemicals (Tokyo, Japan). Ultrapure HNO<sub>3</sub> (68%) was purchased from TAMA Chemicals (Tokyo, Japan). Milli-Q water (18.2 M $\Omega$ · cm) was used to prepare required solutions. The AG MP-1 M resin (100–200 mesh, chloride form) was purchased from Bio-Rad Laboratories (Hercules, CA, USA). For SF-ICP-MS measurement, <sup>242</sup>Pu (CRM 130, plutonium spike assay and isotopic standard, New Brunswick Laboratory, USA) and

<sup>233</sup>U (AEA Technology, 92/233/23, Institute for Reference Materials and Measurements, Geel, Belgium) were used as yield tracer and internal standard, respectively.

# **Digestion and co-precipitation**

The chemical procedures are presented in Fig. 1. Two different kinds of samples, a sample of about 1 g stainless steel (304L) or a mixed sample of about 1 g stainless steel (304L) and 0.5 g concrete (hereafter called the 1.5 g steel + concrete mixed sample), were investigated in this study. The steel sample or steel + concrete mixed sample was transferred to a 250 mL Teflon beaker and spiked with 0.57 pg <sup>242</sup>Pu. For total digestion, 20 mL conc. HCl, 5 mL conc. HNO<sub>3</sub> and 5 g NH<sub>4</sub>HF<sub>2</sub> were used. After covering the Teflon beaker with a Teflon lid, the sample and acids were heated at 200 °C for 30 min on a hot plate. Subsequently, the sample solution was transferred to a 250 mL centrifuge tube. The Teflon beaker was rinsed three times using 5 mL Milli-Q water and this rinse water was added to the tube. The sample solution and rinse water were then diluted to 100 mL using Milli-Q water. After addition of 3 mL 20% TiCl<sub>2</sub> to adjust Pu to Pu (III) and Np to Np (IV), 100 mg Ca<sup>2+</sup>, 100 mg La<sup>3+</sup>, and 3 g NH<sub>4</sub>HF<sub>2</sub> were added into the centrifuge tube to co-precipitate Pu and Np by CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation after shaking. The supernatant was discarded after centrifugation at 3000 rpm for 15 min. Finally, ca. 0.5 g H<sub>3</sub>BO<sub>3</sub> and 20 mL of 7.2 M HNO<sub>3</sub> were added to dissolve the fluoride precipitate, and the sample solution was transferred to a 50 mL centrifuge tube after filtration.

# AG MP-1 M resin column chromatographic separation

The valences of Pu and Np were again adjusted to Pu(III) and Np(IV) by adding 0.3 g ascorbic acid along with a small amount of residual Fe, waiting for about 3 min to ensure complete valence adjustment. Subsequently, 0.3 g NaNO<sub>2</sub> was added to the sample solution followed by heating at 40 °C for 30 min to oxidize Pu(III) to Pu(IV). After that, a 2 mL AG MP-1 M resin column was preconditioned with 20 mL of 7.2 M HNO<sub>3</sub>-0.4 M NaNO<sub>2</sub>, and the sample was loaded onto this column followed by washing the centrifuge tube twice with 5 mL of 7.2 M HNO<sub>3</sub> and loading the washes onto the column. To remove the interfering elements, 20 mL conc. HCl-0.3 mL conc. HNO<sub>3</sub> and 60 mL of 7.2 M HNO<sub>3</sub> were used to rinse the resin. Afterwards, 10 mL of 10 M HCl was used to wash out the residual HNO<sub>3</sub> and remove the Th and U completely. Finally, 20 mL of conc. HBr was used to elute Pu and Np from the resin and the eluate was collected in a 50 mL Teflon beaker.

#### SF-ICP-MS measurement

The eluate was evaporated to dryness at 250  $^{\circ}$ C on a hot plate to remove HBr, then 1 mL TAMA pure HNO<sub>3</sub> was

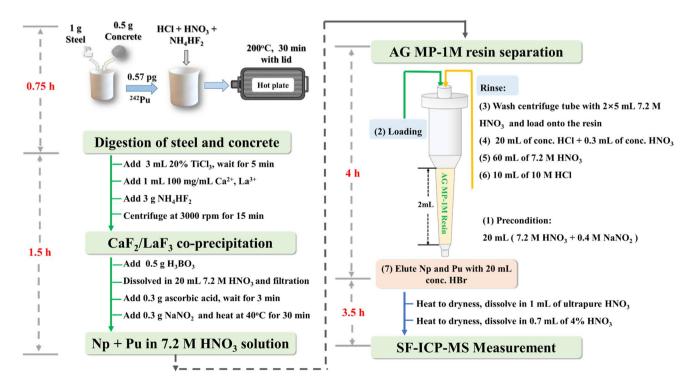


Fig. 1 Flow chart of the developed method

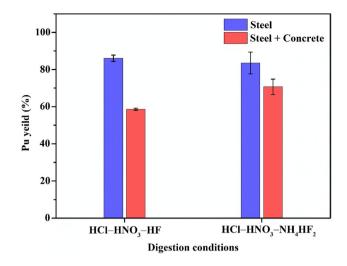
added to dissolve the residue and the solution was heated to near dryness. After that 0.7 mL of 4% HNO<sub>3</sub> with 0.0022 ng mL<sup>-1 233</sup>U as internal standard was used to dissolve the sample. Finally, the Np and Pu isotopes were determined with a SF-ICP-MS instrument (Element XR, Thermo Fisher Scientific Inc., Germany). To improve the sensitivity of SF-ICP-MS, a high efficiency sample introduction system of APEX-Q/ACM (Elemental Scientific Inc, USA) was used, and operation was in a low-resolution mode ( $m/\Delta m = 300$ ) [12]. The detailed parameter settings of the SF-ICP-MS instrument and the sample introduction system have been described elsewhere [12, 13, 22].

# **Results and discussion**

#### Digestion of steel and concrete to release Np and Pu

Stainless steel 304L with high temperature resistance and low corrosion rate has been commonly used in light water reactor nuclear power plants and site buildings [26-28]. The steel and concrete samples contaminated by a nuclear accident and from nuclear facilities undergoing decontamination or decommission have the following characteristics: 1) lots of metal ions will be released after complete digestion of steel samples and they will affect the separation efficiency of resin column chromatography; 2) steel samples from building materials will inevitably have small amounts of concrete attached; and 3) the activity concentrations of Pu and Np in the samples after decontamination or decommission are much higher than their environmental level. Therefore, two different kinds of samples, 1 g of steel and 0.5 g of concrete mixed with 1 g of steel (1.5 g steel + concrete mixed sample), were prepared to explore the digestion effects of different kinds of steel and concrete samples.

The  $HCl - HNO_3 - NH_4HF_2$  digestion system was used to completely digest steel and concrete samples, and the HCl-HNO<sub>3</sub>-HF digestion system was also introduced for comparison. For HCl-HNO<sub>3</sub>-HF acid digestion, 20 mL of conc. HCl, 5 mL of conc. HNO<sub>3</sub>, and 5 mL conc. HF were chosen according to the report by Maxwell et al. [10]. For HCl-HNO<sub>3</sub>-NH<sub>4</sub>HF<sub>2</sub> acid digestion, 20 mL of conc. HCl, 5 mL of conc. HNO<sub>3</sub>, and 5 g of NH<sub>4</sub>HF<sub>2</sub> were chosen, where about 5 g of NH<sub>4</sub>HF<sub>2</sub> was chosen according to the molar concentration of F<sup>-</sup> in HF (28 M, 5 mL) and the mass ratio (4:1) of  $NH_4HF_2$  and sample [19, 21]. The results of <sup>242</sup>Pu yield after using the two different acid digestion conditions are shown in Fig. 2. For the digestion of steel samples, the use of NH<sub>4</sub>HF<sub>2</sub> resulted in similar chemical recoveries of Pu ( $84 \pm 6\%$ , n = 3) to that of using HF  $(86 \pm 2\%, n = 3)$ . For the digestion of steel + concrete mixed samples, the use of NH<sub>4</sub>HF<sub>2</sub> in place of HF presented better chemical recoveries of Pu  $(71 \pm 4\%, n=3)$  than when using



**Fig. 2** The Pu yields with different digestion conditions (n=3)

HF (59  $\pm$  1%, n=3). These findings indicate that NH<sub>4</sub>HF<sub>2</sub> can not only replace HF, but it also provides better digestion than HF for steel + concrete mixed samples. Therefore, 5 g of NH<sub>4</sub>HF<sub>2</sub> combined with 20 mL of conc. HCl and 5 mL of conc. HNO<sub>3</sub> at 200 °C were applied to digest steel + concrete mixed samples.

In digestion, the steel began to be dissolved when the acids were added to the Teflon beaker, and dissolution speed was significantly improved via heating. A 1 g steel sample was dissolved completely by the time the temperature reached 200 °C and this process took only 10 min. For 0.5 g of concrete, with the continuous heating at 200 °C, the NH<sub>4</sub>HF<sub>2</sub> fusion and/or HF that was produced by NH<sub>4</sub>HF<sub>2</sub> decomposition would break the Pu-O and Si-O bonds during this process to achieve the total digestion [19, 20], within 30 min. Finally, the sample solution was transferred to a 250 mL centrifuge tube before it had cooled to room temperature completely. Unless that was done a small amount of black substance would sometimes appear on the beaker inner surface. The occurrence of a black substance during steel digestion has also been observed in previous studies [4, 29], and presented insignificant influence on the Pu recovery, but its composition needs further study.

#### Elimination of matrix effect and interference

Stainless steel (type 304L) chosen in this work, consists of 17.5–20% chromium, 8–11% nickel, <0.08% carbon, <2% manganese, <1% silicon, <0.045% phosphorus, <0.03% sulfur, balance iron [10, 30]; and these elements, especially the iron, will be released after complete digestion and will affect the separation efficiency of resin column chromatography. In addition, elements including U, Pb, Bi, Tl, Hg and Pt from the matrix can cause polyatomic interferences at the same m/z for the SF-ICP-MS measurement of Np and

Pu [12, 13, 17]. Therefore, the CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation and AG MP-1 M resin column separation were introduced to eliminate matrix and interfering elements as much as possible. HF and Ca<sup>2+</sup>/ La<sup>3+</sup> have often been used to get CaF<sub>2</sub>/ LaF<sub>3</sub> coprecipitation [10, 12, 23], in which about 70% of the Fe and 60% of the U can be removed [23]. However, it has been found in previous studies that NH<sub>4</sub>HF<sub>2</sub> could release  $F^-$  in  $H^+$  solution [12, 19]. Therefore,  $NH_4HF_2$  can also replace toxic HF to get CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation. In this work, although F<sup>-</sup> was already present in the digestion solution, a small amount of  $F^-$  will still be lost as SiF<sub>4</sub> gas because the Teflon beaker was only covered by a Teflon lid [19]. In addition, it also was found here that the concentration of F<sup>-</sup> should be at least 2 M, otherwise CaF<sub>2</sub>/ LaF<sub>3</sub> co-precipitate could not form sufficiently. Therefore, an additional 3 g of NH<sub>4</sub>HF<sub>2</sub> was added to ensure the concentration of F<sup>-</sup> was above 2 M in a 100 mL solution. Before the co-precipitation, Pu and Np were reduced to Pu(III) and Np(IV) using 3 mL of 20% TiCl<sub>3</sub> to ensure complete coprecipitation of Pu and Np [6]. In this step, some fraction of Fe was also reduced to Fe<sup>2+</sup>, and the redox-couple Fe<sup>3+</sup>/  $Fe^{2+}$  as a redox buffer in solution will protect Np(IV) [11, 31]. Furthermore, the 2 mL AG MP-1 M resin column was chosen to separate and purify Np and Pu from the sample solution [12-14]. The Fe and other matrix elements were further removed during the loading and separation processes [32, 33], where these matrix elements have a tiny influence on the separation efficiency of Pu in the resin column, as shown in Fig. 2; after HCl-HNO<sub>3</sub>-NH<sub>4</sub>HF<sub>2</sub> digestion, high chemical recovery of  ${}^{242}$ Pu ( $84 \pm 6\%$ , n=3) in the steel samples was obtained, and the chemical recovery of <sup>242</sup>Pu  $(71 \pm 4\%, n = 3)$  in steel + concrete mixed samples was only decreased by ca.13%, which was less than the loss using the HCl-HNO<sub>3</sub>-HF digestion (ca. 27%). For <sup>238</sup>U, which is ubiquitous, the concentrations were  $< 10 \text{ pg mL}^{-1}$  in the final sample solution (0.7 mL) after CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation and AG MP-1 M resin column separation, and the decontamination factor (DF) of <sup>238</sup>U was estimated to be about  $1.1 \times 10^6$  for 0.5 g of the in-house standard reference material (from the Sea of Japan in Ishikawa Prefecture). The details on eliminating matrix and interfering elements have been reported elsewhere [12, 13]. In this work, the lowresolution mode ( $m/\Delta m = 300$ ) and the APEX-Q/ACM high

efficiency sample introduction system were used to maximize the instrument sensitivity (about  $3 \times 10^7$  cps ppb<sup>-1</sup>). For the SF-ICP-MS measurement, the formation rate of  $^{238}$ UH<sup>+</sup>/U<sup>+</sup> and the ratio of  $^{237}$ Np/U<sup>+</sup> due to the U tailing effect were all in the level of  $10^{-5}$ ; therefore, there were only several cps contributions for  $^{239}$ Pu and  $^{237}$ Np from U, which can be easily corrected.

# Simultaneous determination of <sup>237</sup>Np and Pu isotopes in steel and concrete

To verify whether <sup>242</sup>Pu is suitable as the yield tracer for the simultaneous determination of Np and Pu isotopes in steel and concrete samples by SF-ICP-MS after HCl-HNO<sub>3</sub>-NH<sub>4</sub>HF<sub>2</sub> total digestion, CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation, and AG MP-1 M resin column separation, the same samples (1 g steel sample or 1.5 g steel + concrete mixed sample) with spiking by an in-house  ${}^{237}Np + {}^{242}Pu$ mixed standard solution (0.405 pg  $^{237}$ Np and 0.285 pg  $^{242}$ Pu) were studied following the chemical procedure presented in Fig. 1. The results are listed in Table 1. The average vield ratios of Np/Pu were  $0.922 \pm 0.025$  and  $0.948 \pm 0.006$ using NH<sub>4</sub>HF<sub>2</sub> for steel samples and steel + concrete samples, respectively, and similar yield ratios of <sup>237</sup>Np/<sup>242</sup>Pu were obtained using HF  $(0.929 \pm 0.017$  for steel samples;  $0.928 \pm 0.002$  for steel + concrete samples), suggesting that approximately 5-8% more Np get lost in the chemical separation procedure compared with <sup>242</sup>Pu. Compared to use of the AG MP-1 M resin column for the purification of Np and Pu isotopes in soil, sediment, and urine samples [12 - 14], the loss of Np in the present study was mainly attributed to the metal ions (e.g., Fe) that affect the separation efficiency of Np during AG MP-1 M resin column chromatography, since large amounts of metal ions were produced after steel digestion; for example, ca. 600 mg of Fe was released from 1 g of steel. Although most of the metal ions are removed by the CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation and they go through the resin column during the sample loading process [23, 33], the high amount of Fe can compete with trace Np and Pu isotopes in the CaF<sub>2</sub>/LaF<sub>3</sub> co-precipitation and column chromatography processes, resulting in tiny fractionation between Np and Pu. Therefore, the results of Np should be corrected as described by Eq. (1):

Table 1	The <sup>242</sup> Pu and <sup>237</sup> Np					
yields obtained using the						
developed method						

Sample	Digestion conditions	Yield (%) (r	i=3)	Average Np/Pu
		<sup>242</sup> Pu	<sup>237</sup> Np	(yield ratio, $n=3$ )
1 g steel	HCl-HNO <sub>3</sub> -HF	82±5	76±4	$0.929 \pm 0.017$
	HCl-HNO3-NH4HF2	$78 \pm 1^{*}$	$72 \pm 2^*$	$0.922 \pm 0.025$
1 g steel + 0.5 g concrete	HCl-HNO3-HF	$56\pm7$	$52\pm 6$	$0.928 \pm 0.002$
(mixed sample)	HCl-HNO <sub>3</sub> -NH <sub>4</sub> HF <sub>2</sub>	$64 \pm 3$	$61 \pm 3$	$0.948 \pm 0.006$

\*This yield of Np or Pu was calculated for two samples (n=2)

$$(Np)_{corrected} = \frac{(Np)_{measured}}{(Np/Pu)_{yield\ ratio}}$$
(1)

where  $(Np)_{measured}$  is the measured counting rate (cps). (Np/Pu) <sub>yield ratio</sub> is the average yield value of  $^{237}Np/^{242}Pu$  that is obtained under the corresponding conditions of sample and digestion.

### **Evaluation of the developed method**

To assess the accuracy of the developed method, 1 g steel and 0.5 g in-house sediment standard reference material (from the Sea of Japan in Ishikawa Prefecture) spiked with 0.57 pg <sup>242</sup>Pu were placed into a 250 mL Teflon beaker. This sediment reference sample was pre-ignited at high temperature (1000 °C) to prepare Pu containing refractory material. The overall sample preparation procedure was as illustrated in Fig. 1, and the obtained results were as listed in Table 2. For the Pu isotopes, the average activities of <sup>239</sup>Pu, <sup>240</sup>Pu, and <sup>240+239</sup>Pu were  $0.561 \pm 0.017$ ,  $0.446 \pm 0.001$ , and  $1.008 \pm 0.018$  mBq g<sup>-1</sup>, which agreed well with the respective reference values of  $0.569 \pm 0.061$ ,  $0.437 \pm 0.029$ , and  $1.006 \pm 0.068$  mBq g<sup>-1</sup>. The average atom ratio of  $^{240}$ Pu/ $^{239}$ Pu was measured as  $0.217 \pm 0.007$ , which also agreed well with the reference value of  $0.209 \pm 0.009$ . The average activity of  $^{237}$ Np was measured to be  $0.268 \pm 0.022$  $\mu$ Bq g<sup>-1</sup>, which also agreed well with the reference value of  $0.268 \pm 0.031 \ \mu\text{Bg g}^{-1}$ , and the average atom ratio of  $^{237}$ Np/ $^{239}$ Pu was measured as 0.043 ± 0.005, also in good agreement with the reference value of  $0.044 \pm 0.003$ . Therefore, it was concluded that the developed method can be used to accurately determine <sup>237</sup>Np and Pu isotopes in environmental samples and nuclear power plant decommissioning samples containing high refractory materials.

For the SF-ICP-MS measurement, the corresponding parameters were set according to Huang et al. [12], and the instrumental sensitivity was about  $3 \times 10^7$  cps ppb<sup>-1</sup>. Based on the counting rates of <sup>237</sup>Np and Pu isotopes and chemical recovery of <sup>242</sup>Pu (85%), the limits of detection

(LODs) of <sup>237</sup>Np, <sup>239</sup>Pu, and <sup>240</sup>Pu were calculated for 1.5 g steel + concrete mixed samples to be 0.25 fg g<sup>-1</sup> ( $6.5 \times 10^{-6}$  mBq g<sup>-1</sup>), 0.16 fg g<sup>-1</sup> ( $3.7 \times 10^{-4}$  mBq g<sup>-1</sup>), 0.14 fg g<sup>-1</sup> ( $1.2 \times 10^{-3}$  mBq g<sup>-1</sup>), respectively [34]. The LODs of Pu isotopes were lower than that reported by Wang et al.[17] due to the high sensitivity of the present measurement system. The LOD of <sup>237</sup>Np was also corrected for chemical fractionation, and it was the same level as the 0.23 fg g<sup>-1</sup> LOD obtained by Huang et al. [12]. These LODs were sufficiently low enough to analyze Np and Pu isotopes for steel and concrete samples during decommission and decontamination of a nuclear reactor.

The whole analytical process can be achieved within 1 day for 24 samples with the support of a 24—hole vacuum box system as shown in Fig. 1, including 0.75 h for digestion and transfer of the samples, 1.5 h for  $CaF_2/LaF_3$  co-precipitation and filtration, 4 h for chromatography and sample separation, and 3.5 h for SF-ICP-MS sample preparation and measurement. Therefore, it was concluded that this method can analyze great numbers of steel and concrete samples rapidly after a nuclear accident or during the decommission and decontamination of a nuclear reactor.

# Conclusions

In this study, a novel, simple, and rapid method was developed for the simultaneous analysis of <sup>237</sup>Np and Pu isotopes in steel and concrete samples.  $NH_4HF_2$  in combination with conc. HCl and conc. HNO<sub>3</sub> was chosen for the total digestion of the steel and concrete samples. Then,  $CaF_2/LaF_3$ co-precipitation was used to remove matrix elements from the samples, and  $NH_4HF_2$  was also used to get  $CaF_2/LaF_3$ co-precipitation. Afterwards, an anion exchange resin (AG MP-1 M) column was employed to separate and purify Np and Pu. Finally, the <sup>237</sup>Np and Pu isotopes were determined by the SF-ICP-MS measurement system equipped with an APEX-Q/ACM sample introduction system. The method avoids the use of toxic HF, and can be completed within

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Table 2 Analytical results for <sup>237</sup>Np and Pu isotopes in stainless steel with addition of sediment that has been treated at high temperature

Sample ID	Steel (g)	Sediment (g)	$^{237}$ Np activity (µBq g <sup>-1</sup> )	<sup>237</sup> Np/ <sup>239</sup> Pu atom ratio	<sup>239</sup> Pu activity (mBq g <sup>-1</sup> )	<sup>240</sup> Pu activity (mBq g <sup>-1</sup> )	<sup>240+239</sup> Pu activity (mBq g <sup>-1</sup> )	<sup>240</sup> Pu/ <sup>239</sup> Pu atom ratio
1	0.996	0.506	$0.294 \pm 0.072$	$0.048 \pm 0.012$	$0.545 \pm 0.034$	$0.447 \pm 0.037$	$0.991 \pm 0.050$	$0.223 \pm 0.018$
2	1.018	0.505	$0.258 \pm 0.067$	$0.040 \pm 0.010$	$0.579 \pm 0.040$	$0.448 \pm 0.041$	$1.027\pm0.057$	$0.210 \pm 0.019$
3	1.023	0.504	$0.253 \pm 0.058$	$0.040 \pm 0.009$	$0.560 \pm 0.032$	$0.445 \pm 0.039$	$1.005\pm0.051$	$0.216 \pm 0.019$
Average $\pm$ SD			$0.268 \pm 0.022$	$0.043 \pm 0.005$	$0.561 \pm 0.017$	$0.446 \pm 0.001$	$1.008 \pm 0.018$	$0.217 \pm 0.007$
Reference value	e(n=3)		$0.268 \pm 0.031$	$0.044 \pm 0.003$	$0.569 \pm 0.061$	$0.437 \pm 0.029$	$1.006\pm0.068$	$0.209 \pm 0.009$

The value of  $^{237}$ Np has been corrected using Eq. (1)

SD standard deviation

1 day. The decontamination factor of U was estimated to be  $1.1 \times 10^6$ . However, it should be noted that the average yield ratios of  ${}^{237}$ Np/ ${}^{242}$ Pu were  $0.922 \pm 0.025$  and  $0.948 \pm 0.006$ for the analysis of steel samples and steel + concrete mixed samples, respectively, indicating that about 5 - 8% more <sup>237</sup>Np was lost compared with <sup>242</sup>Pu tracer because of the high iron amount, therefore, the results of Np need to be corrected according to the average yield ratio of <sup>237</sup>Np/ <sup>242</sup>Pu. The accuracy of this developed method for the Pu and Np determination was validated by analyzing a mixed sample of steel + sediment with known amounts of <sup>237</sup>Np and Pu isotopes that had been treated at high temperature (1000°C), and the determined <sup>239</sup>Pu, <sup>240</sup>Pu and <sup>237</sup>Np concentrations agreed well with the reference values. In addition, the low LODs obtained for <sup>237</sup>Np, <sup>239</sup>Pu, and <sup>240</sup>Pu of 0.25, 0.16, and 0.14 fg  $g^{-1}$ , respectively, will allow for effective detection of low amounts of Pu and Np in steel and concrete samples.

Acknowledgements This study was supported by the JSPS KAKENHI (JP 17K00537 and JP 21H03609), the Grant of Fukushima Prefecture related to research and development in radiological sciences, and ERAN I-21-24. Shuai Zhang thanks the China Scholarship Council for offering a scholarship (201906190116) to support his Ph. D. study

Author contributions All authors contributed to the study conception and design. Material preparation, data collection and analysis were performed by [Shuai Zhang], [Guosheng Yang], [Jian Zheng]. The draft of the manuscript was written by [Shuai Zhang] and all authors commented on previous versions of the manuscript. All authors read and approved the final manuscript.

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