Solvent extraction of Pu(IV) with TODGA in C_6 mimTf₂N

Xiaohong Huang • Qiuyue Zhang • Jinping Liu • Hui He • Wenbin Zhu • Xiaorong Wang

Received: 9 July 2012 / Published online: 23 April 2013 © Akadémiai Kiadó, Budapest, Hungary 2013

Abstract Studies on the solvent extraction of Plutonium (Pu(IV)) from aqueous nitric acid by N, N, N', N' -tetraoctyldiglycolamide (TODGA) in 1-hexyl-3-methylimidazolium-bis(trifluoromethylsulfonyl) imide $(C_6$ mimTf₂N) room temperature ionic liquid (RTIL) were carried out. It was found that $Pu(IV)$ is extracted into RTIL phase as $[Pu(NO₃)(TOD G(A)$]³⁺ through cation exchange mechanism. Extraction reaction equation is obtained by the influence of acidity and extractant concentration, and the parameters of thermodynamic equilibrium constant was calculated.

Keywords Solvent extraction \cdot Plutonium(IV) \cdot TODGA \cdot C_6 mimTf₂N

Introduction

Recently, room temperature ionic liquids (RTILs), especially these containing alkylimidazolium cations, such as 1-hexyl-3-methylimidazolium bis(trifluoromethyl)sulphonyl-imide(C_6 mimTf₂N) have received increasing attentions as next generation diluents for extractions in nuclear fuel reprocessing. A result of their unique physical and chemical properties [\[1](#page-4-0), [2\]](#page-4-0). A preliminary assessment showed that the relative viscosity of 1-octyl-3-methylimidazolium $bis(trifluorometry)$ sulphonyl-imide C_8 mimTf₂N ionic liquid is larger than C_6 mimTf₂N ionic liquid [\[3](#page-4-0)]. Moreover the extractive ability of C_6 mimTf₂N to Pu(IV) is higher than 1-butyl-3-methylimidazolium bis(trifluoromethyl)sulphonyl-imide (C_4 mimTf₂N) ionic liquid [\[2](#page-4-0)]. Based on the possible stability towards radiation according to the previous research [\[4](#page-4-0)], RTILs are expected to be used in the reprocessing of spent fuel as diluent or extractant [[5,](#page-4-0) [6](#page-4-0)]. There is big difference between the extraction behavior of metal ions in ionic liquid and that in the molecular organic solvent system. Thus, it is necessary to investigate the solvent extraction of lanthanides and actinides in water/ RTIL system. Moreover there are many research reports about the extraction of actinides, lanthanides and strontium (Sr), cesium (Cs) by tri-butyl-phosphate (TBP)/n-octyl (phenyl)-N,N-diisobutylcarbamoylmethylphosphine oxide (CMPO)/crown ether, using ionic liquid C_n mimTf₂N $(n = 4, 6, 8)$ as the diluent $[1, 7-10]$ $[1, 7-10]$. In comparison, the extraction ratio of one metal ion by TBP is slow, the solubility of CMPO in ionic liquid is low, crown ether is usually used to extract Sr and Cs. TODGA (Fig. [1\)](#page-1-0) as a class of actinides novel extractant, synthesis is simple, low cost, resistance to radiation, not easy hydrolysis, degradation products do not affect the extraction process and overcomes the shortcoming of solubility low compared with CMPO in RTILs [[11–](#page-4-0)[19\]](#page-5-0). Solvent extraction studies on lanthanides and actinides from aqueous solutions suggest that TODGA dissolved in kerosene greatly enhances the extractability of lanthanides and actinides [\[17–23](#page-5-0)]. While at the same experimental condition of americium(Am(III)) and europium(Eu(III)), plutonium exists as Pu(IV), which is the reason for the extraction study of Pu(IV) by TODGA because no research is reported in this area. Here, we chose a hydrophobic ionic liquid— C_6 mimTf₂N (Fig. [1](#page-1-0)) as a diluent, using TODGA for the solvent extraction studies of Pu(IV) in the present work for accumulating the basic data for the effective use of this type of RTILs for the solvent extraction of actinides.

X. Huang \cdot Q. Zhang \cdot J. Liu \cdot H. He (\boxtimes) \cdot W. Zhu \cdot X. Wang Department of Radiochemistry, China Institute of Atomic Energy, Beijing 102413, People's Republic of China e-mail: hehui@ciae.ac.cn

Experimental

Reagents and instruments

All chemicals were regent grade or higher. The TODGA was synthesized in our laboratory with a purity over 98 %. C_6 mimTf₂N was synthesized and purified according to the method described in paper $[24]$ $[24]$ and its purity was $>99\%$ checked with [\[1](#page-4-0)] H and [\[13](#page-4-0)] C NMR spectral data, TLC, and mass spectrometry.

Stock solution of [\[9](#page-4-0), [23\]](#page-5-0) Pu was purified with 256×4 pyridine-type resin (the length of the column is 350 mm, with the diameter 6 mm)through adsorption in 8 M $HNO₃$ solution and eluate in 0.5 M HNO₃, and purity was measured by alpha spectrometry. Pu in $0.5-1$ M HNO₃ was adjusted to Pu(IV) by using liquid N_2O_4 . Required amount of Pu (about $10 \mu g/mL$) was transferred into the equilibration tube containing aqueous phase and its valency was adjusted to Pu(IV) by N_2O_4 liquid. Because in nitric acid solution, the main valence state of plutonium includes III, IV and VI. N_2O_4 oxidizes Pu(III) to Pu(IV) and reduces Pu(VI) to Pu(IV), so it can be used for the valence adjustment of plutonium as Pu(IV), which can be proved by the extraction of TTA/xylene and then measured by the a spectrometry. Reaction equations as follows:

$$
N_2O_4 + H_2O \rightarrow HNO_3 + HNO_2 \tag{1}
$$

$$
HNO2 + Pu3+ + H+ \rightarrow Pu4+ + NO \uparrow + H2O
$$
 (2)

$$
NO_2^- + PuO_2^{2+} + 2H^+ \rightarrow Pu^{4+} + NO_3^- + H_2O
$$
 (3)

TODGA in RTILs, were equilibrated with the given concentration of $HNO₃$ three times before the extraction of Pu(IV) in the same concentration of $HNO₃$.

An LS-600LL liquid scintillation counter. (Shichema, Japan) was used to measure the concentration of Pu(IV).

Procedure

Equal volumes of ionic liquids containing TODGA and aqueous nitric acid solution containing Pu(IV) were vortexed for a certain time and equilibrated in a shaker bath at required temperature. The mixed solutions were then centrifuged for phase separation, aliquots from both phases were withdrawn for liquid scintillation counting. At 293.15 K, the density of C_6 mimTf₂N ionic liquid is 1.3 times larger than that of water. The viscosity and the density of TODGA are larger than those of water also [\[3](#page-4-0)]. Thus the aqueous phase is at upper position of the solution. From activities of Pu(IV) in initial solution and that of in raffinate, the distribution ratio (D) is calculated as follows:

$$
D = \frac{M_{\text{total}} - M_{\text{remainder}/\text{aq}}}{M_{\text{remainder}/\text{aq}}}
$$

where M_{total} stands for the total concentration of M, namely the metal added and $M_{\text{remainder/aq}}$ means the concentration of M remained in aqueous phase.

The distribution ratio was measured repeatedly over a period of time until it reaches equilibrium. All the measurements were done in duplicate and D values obtained were within ± 5 % with good material balance (\geq 95).

Results and discussion

The effect of extraction time

0.1 mol/L TODGA/ C_6 mimTf₂N were selected to extract $HNO₃$ and tracer amount of Pu(IV) in the aqueous phase in which the $HNO₃$ concentration was 0.988 and 3.93 mol/L respectively. The effect of extraction time on the extraction at different conditions were shown in the Fig. 2. The equilibrium was established after 1 min of contact. For the sake of reaching efficient equilibrium, the distribution ratio were all measured after 10 min of vortexing.

Fig. 2 Equilibrium time of $HNO₃$ and Pu(IV). Organic phase 0.1 M TODGA/C₆mimTf₂N, *Aqueous phase* 0.988 M HNO₃ (HNO₃) and Trace amount $Pu(IV)$ 3.93 M HNO₃ respectively

Effect of HNO₃ concentration on $D_{(\text{HNO3})}$ and $D_{\text{Pu(IV)}}$

Figure 3 gives distribution ratios of $HNO₃$ as a function of [HNO₃] at 0.1 M TODGA/C₆mimTf₂N. $D_{(HNO_3)}$ remains almost ≤ 0.15 with 0.34 M up to 7.5 M HNO₃. At nitric acid concentration higher than 1.8 M, the $D_{(HNO_3)}$ values increased with increase of nitric acid concentration.

Figure 4 gives the plot of distribution ratios of $Pu(IV)$ as a function of $[HNO_3]$ at 0.1 M TODGA/C₆mimTf₂N and 0.1 M TODGA/n-dodecane. The results indicate that the distribution ratio of Pu(IV) in ionic liquid is much higher than that of in the system of n -dodecane.

Figure 5 displays log–log plot of distribution ratios of Pu(IV) as a function of $[HNO₃]$ at 0.1 M TODGA– C_6 mimTf₂N. From the slope of the plot in Fig. 5, it can be seen that in the acidity range studied, the slope of 0.85 close to 1, which suggests that the molar ratio of NO_3^- and Pu(IV) in the complex is 1:1, the stoichiometry of the

Fig. 3 Dependence of distribution ratios of $HNO₃$ versus $HNO₃$ concentration. Organic phase 0.1 M TODGA/C₆mimTf₂N, Aqueous phase HNO₃

Fig. 4 Dependence of distribution ratios of Pu versus $HNO₃$ concentration. Organic phase 0.1 M TODGA/C₆mimTf₂N, 0.1 M TODGA/n-dodecane respectively. Aqueous phase trace amount $Pu(IV) + HNO₃$

Fig. 5 Dependence of distribution ratios of Pu(IV) on HNO₃ concentration. Organic phase 0.1 M TODGA/C₆mimTf₂N, Aqueous *phase* trace amount $Pu(IV) + HNO₃$

cationic complex transferred into the IL phase is $[Pu(NO₃)(TODGA)_x]$ ³⁺.

Extraction mechanism

Figures 6, [7,](#page-3-0) [8](#page-3-0) and [9](#page-3-0) show plots of distribution ratios of Pu(IV) in C_6 mimTf₂N as a function of TODGA concentration at different concentration of $HNO₃$. The results show that the slopes of $log(D_{Pu(IV)})$ vs $log(TODGA)$ change from 0.8 to 1.09, which can be considered as 1 approximately. Therefore, in the aqueous acidity studied, the extraction reaction equation can be written as follows:

$$
Pu_{aq}^{4+} + NO_{3aq}^- + [TODGA]_{IL}^- + 3[C_6min]_{IL}^+
$$

\n
$$
\Leftrightarrow [Pu(NO_3)(TODGA)]_{IL}^{3+} + 3[C_6min]_{aq}^+
$$
 (4)

The equilibrium constant can be expressed as $[25, 26]$ $[25, 26]$ $[25, 26]$ $[25, 26]$:

Fig. 6 TODGA dependency of distribution ratios of Pu(IV) in C_6 mimTf₂N. Organic phase 0.1 M TODGA/ C_6 mimTf₂N, Aqueous *phase* trace amount $Pu(IV) + 0.089 M HNO₃$

Fig. 7 TODGA dependency of distribution ratios of Pu(IV) in C_6 mimTf₂N. Organic phase 0.1 M TODGA/ C_6 mimTf₂N, Aqueous *phase* trace amount $Pu(IV) + 0.52 M HNO₃$

Fig. 8 TODGA dependency of distribution ratios of Pu(IV) in C_6 mimTf₂N. Organic phase 0.1 M TODGA/ C_6 mimTf₂N, Aqueous *phase* trace amount $Pu(IV) + 1.81$ M $HNO₃$

$$
\beta = \frac{\{[Pu(NO_3)(TODGA)]^{3+}\}}{[Pu^{4+}][NO_3^-][TODGA]} \cdot \frac{f_{\{[Pu(NO_3)(TODGA)]^{3+}\}}}{f_{[Pu^{4+}]}\cdot f_{[NO_3^-]} \cdot f_{[TODGA]}}
$$
(5)

here, [M] represents molar concentration (mol/L), and $f_{[M]}$ means the activity coefficient. According to the theory of Debye–Huckel [\[27](#page-5-0)], as a first approximation, the activity coefficient of dilute solution is correlative with ionic strength only. When the concentration of individual ion is moderate, Davies equation can be used for the calculation of activity coefficients, which can be written as follows:

$$
-\log f_{z\pm} = AZ^2 \left[\frac{\sqrt{I}}{1+\sqrt{I}} - 0.2I\right]
$$
 (6)

Fig. 9 TODGA dependency of distribution ratios of Pu(IV) in C_6 mimTf₂N. Organic phase 0.1 M TODGA/ C_6 mimTf₂N, Aqueous phase trace amount $Pu(IV) + 2.94$ M HNO₃

$$
I = 0.5 \times (C_1 Z_1^2 + C_2 Z_2^2 + C_3 Z_3^2 + \cdots) = 0.5 \times \sum C_I Z_I^2
$$
\n(7)

here I is the ionic strength in molality, C is the molar concentration of ion in mol/L and Z is the charge of it. As for aqueous solution, the constant A is 0.509 at room temperature and about 0.52 at the temperature of (28 ± 0.5) °C. Nitric acid solution of 0.988 M, with the Pu(IV) concentration 5.8×10^{-8} M, I is calculated as follows:

$$
I = 0.5 \times (0.988 \times 1^{2} + 0.988 \times 1^{2} + 5.8 \times 10^{-8} \times 4^{2})
$$

= 0.988

so we get

$$
- \log f_{Z\pm} = 0.52 \times Z^2 \times (0.4985 - 0.1976) = 0.156Z^2
$$

$$
f_{\text{[NO_3^-]}} = 10^{-0.156}, f_{\text{[Pu(NO_3)(TODGA)]}^{3+}}
$$

$$
= 10^{-0.156 \times 9}, f_{\text{[TODGA]}} = 1, f_{\text{[Pu^{4+}]}} = 10^{-0.156 \times 16}
$$

And the activity coefficient $f = \frac{f_{\{ [\text{Pu}(\text{NO}_3)(\text{TDDGA})]^3+ \}}}{f_{[\text{Pu}^{4+}]}f_{[\text{NO}_3^-]}f_{[\text{TDDGA}]}}$ $= 17.701.$

In general, K can be represented as

$$
K = \frac{[Pu(NO_3)(TODGA)^{3+}]}{[Pu^{4+}][NO_3][TODGA]}
$$
 (8)

here K is the equilibrium constant. Pu(IV) in aqueous solution exists as $Pu(NO_3)^{3+}$, $Pu(NO_3)^{2+}$ etc. If C_{Pu} is the total concentration of plutonium in the aqueous phase and β_I is the stepwise formation constant between Pu(IV) and NO_3^- , C_{Pu} can be expressed as

$$
\mathcal{C}_{Pu} = [Pu^{4+}](1 + \sum \beta_I [NO_3^{-}]^I)
$$

Table 1 TODGA dependency of equilibrium constant and thermodynamic equilibrium constant

[TODGA] M $D - D_0$ $log K$ K			
0.01		6.49×10^{2} 5.24 1.74×10^{5} 3.09 $\times 10^{6}$	
0.05		1.18×10^3 4.80 6.34 $\times 10^4$ 1.12 $\times 10^6$	
0.08		1.87×10^3 4.80 6.29 $\times 10^4$ 1.11 $\times 10^6$	
0.1		3.70×10^3 5.00 9.95×10^4 1.76 $\times 10^6$	
0.12		4.20×10^3 4.97 9.41×10^4 1.67×10^6	

So the Eq. [\(8](#page-3-0)) can be written

$$
K = \frac{\left\{ \left[Pu(NO_3)(TODGA) \right]^{3+} \right\} (1 + \sum \beta_I [NO_3^-]^I)}{CPu[NO_3^-][TODGA]}
$$
(9)

The distribution ratio:

$$
D = \frac{\left\{ \left[Pu(NO_3)(TODGA) \right]^{3+} \right\}}{C_{Pu}} \tag{10}
$$

Inserting (10) into (9) gives

$$
K = \frac{(D - D_0)(1 + \sum \beta_I [NO_3^-]^I)}{[NO_3^-][TODGA]}
$$
\n(11)

where D and D_0 denote the distribution ratios of Pu(IV) in organic phase of TODGA–RTIL and RTIL, respectively.

 $(1 + \sum \beta_I [NO_3^-]^I)$ is 2.69 ± 0.14 in 1 M [HNO₃], which was taken from the literature [[20\]](#page-5-0). So, we obtain

$$
K = \frac{2.69 \times (D - D_0)}{[NO_3^-][\text{TODGA}]}
$$

and

 $\log K = 0.43 + \log(D - D_0) - \log[NO_3^-] - \log[TODGA]$

According to the D values obtained at (28 ± 0.5) °C and 1 M [HNO₃], the logK and β values are determined as described in Table 1

In general, the solubility of the neutral complex in aqueous solution is far less than that of in organic solvents due to the polarity. Thus, the neutral complex is easy to be extracted into the organic solvents, while the charged complex is difficult to be transferred into organic phase. For example, the uranium and plutonium are extracted into TBP/OK in the Purex process. However, ionic liquid is a very strong polar organic solvent, which means that the charged complex extracted should be larger for ionic compounds than the neutral complex. The above results are accordable completely with this basis theory [\[28](#page-5-0)].

Conclusion

In the range of experimental conditions, the extraction percentage of Pu(IV) is more than 99 $%$ in 0.1 M TODGA/ C_6 mimTf₂N. Through cation exchange mechanism, Pu(IV) is extracted into RTIL phase, as $[Pu(NO₃)(TODGA)]³⁺$. Extraction reaction equation is obtained by the influence of acidity and extractant concentration, and the parameters of thermodynamic equilibrium constant is calculated.

Acknowledgments Authors thank the NCFC (NO. 91126006) programme for financial support of the project. The authors are also grateful to staffs in Radioecology programme (headed by hehui), Department of RadioChemistry, China Institute of Atomic Energy, for all support in this work.

References

- 1. Alok Rout KA, Srinivasan Venkatesan TG, Vasudeva Rao PR et al (2011) Room temperature ionic liquid diluents for the extraction of Eu(III) using TRUEX extractants[J]. J Radioanal Nucl Chem 290:215–219
- 2. Alok Rout, Venkatesan KA et al (2011) Ionic liquid extractants in molecular diluents: extraction behavior of plutonium(IV) in 1,3 diketonate ionic liquids. Solvent Extr Ion Exch 29:602–618
- 3. Ignatev NV, Biermann UW, Kucheryna A et al (2005) New ionic liquids with tris (perfluoroalkyl) trifluorophosphate (FAP) anions. J Fluor Chem 126:1150–1159
- 4. Allen D, Baston G, Bradely AE et al (2002) An investigation of the radiochemical stability of ionic liquids. Green Chem 4:152
- 5. Heitzman H, Young BA, Rausch DJ et al (2006) Fluorous ionic liquids as solvents for the liquid–liquid extraction of metal ions by macrocyclic polyethers. Talanta 69(2):527–531
- 6. Giridhar P, Venkatesan KA, Srinivasan TG et al (2005) Extraction of uranium(VI) from nitric acid medium by 1.1 M tri-nbutylphosphate in ionic liquid diluent [J]. J Radioanal Nucl Chem 265(1):31–38
- 7. Vasudeva Rao PR et al (2012) Potential applications of room temperature ionic liquids for fission products and actinide separation. Sep Sci Technol 47(2):204–222
- 8. Billard I et al (2009) Actinide and lanthanide speciation in imidazolium-based ionic liquids. Radiochim Acta 97(7):355–359
- 9. Billard I et al (2011) Understanding the extraction mechanism in ionic liquids: $UO_2^{2+}/HNO_3/TBP/C_4$ -mimTf₂N as a case study. Solvent Extr Ion Exch 29(4):577–601
- 10. Ansari SA, Mohapatra PK et al (2011) Extraction of cae $sium(I)$ from $HNO₃$ medium using room temperature ionic liquid containing calyx[4] crown ligands as the selective extractants. Radiochim Acta 99(4):713–717
- 11. Dietz ML, Dzielawa JA (2001) Ion-exchange as a mode of cation transfer into room-temperature ionic liquids containing crown ethers: implications for the ''Greenness'' of ionic liquids as diluents in liquid–liquid extraction. Chem Commun 20:2124– 2125
- 12. Stepinski DC, Jensen MP, Dzielawa JA et al (2005) Synergistic effects in the facilitated transfer of metal ions into room-temperature ionic liquids. Green Chem 7(3):151–158
- 13. Chun SK, Dzyuba SV, Bartsch RA (2001) Influence of structural variation in room-temperature ionic liquids on the selectivity and efficiency of competitive alkali metal salt extraction by a crown ether. Anal Chem 73(15):3737–3741
- 14. Jensen MP, Neuefeind J, Beitz JV et al (2003) Mechanisms of metal ion transfer into room-temperature ionic liquids: the role of anion exchange. J Am Chem Soc 25:15466
- 15. Visser AE, Rogers RD (2003) Room-temperature ionic liquids: new solvent for f-element separations and associated solution chemistry. J Solid State Chem 171:109
- 16. Nakashima K, Kubota F, Maruyama T et al (2003) Ionic liquids as a novel solvent for lanthanide extraction. Anal Sci 19:1097
- 17. Suzuki H, Sasaki Y, Sugo Y et al (2004) Extraction and separation of Am(III) and Sr(II) by N, N, N', N' -tetraoctyl-3oxapentanediamide(TODGA). Radiochimca Acta 92(8):463–466
- 18. Magnusson D, Christiansen B, Glatz JP et al (2007) Partitioning of minor actinides from PUREX raffinate by the TODGA process. Proceedings of Global, Boise, pp. 713–718
- 19. Hérès X, Sorel C, Miguirditchian M et al (2009) Results of recent counter-current tests on An(III)/Ln(III) separation using TODGA extractant. Proceedings of the Global, Paris
- 20. Lohithakshan KV, Aggarwal SK (2008) Solvent extraction of Pu(IV) with CMPO in 1-octyl 3-methyl imidazolium hexa fluorophosphate(C8mimPF6) room temperature ionic liquid (RTIL). Radiochim Acta 96:93–97
- 21. Modolo G et al (2007) Development of a TODGA based process for partitioning of actinides from a PUREX raffinate part I: batch extraction optimization studies and stability tests. Solvent Extr Ion Exch 25(6):703–721
- 22. Modolo G et al (2008) Demonstration of a TODGA-based continuous counter-current extraction process for the partitioning of actinides from a simulated PUREX raffinate part II: centrifugal contactor runs. Solvent Extr Ion Exch 26(1):62–76
- 23. Magnusson D et al (2009) Demonstration of a TODGA based extraction process for the partitioning of minor actinides from a PUREX raffinate part II: centrifugal contactor run using genuine fuel solution. Solvent Extr Ion Exch 27(1):26–35
- 24. Dzyuba SV, Bartsch RA (2001) Efficient synthesis of 1-alkyl(aralkyl)-3-methyl(ethyl) imidazoliumhalides: precursors for room temperature ionic liquids. J Heterocycl Chem 38:265–268
- 25. Klatt LN, Rouseff RL (1970) Analysis of the polarographic method of studying metal complex equilibriums. Anal Chem 42:1234
- 26. Momoki K, Sato H, Ogawa H (1072) Calculation of successive formation constants from polarographic data using a high-speed digital computer. Anal Chem 1967:39
- 27. Davies CW (1962) Ion association. Butterworths, London
- 28. Inczedy J, Sc D (1976) Analytical applications of complex equilibria, Akademiai Kiado, Budapest