DETERMINATION OF ²³⁷Np IN LARGE VOLUME SAMPLES OF SEA WATER BY A RADIOCHEMICAL PROCEDURE

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A radiochemical procedure followed by alpha spectrometry has been developed for the determination of $237Np$ present at low activity concentrations in seawater. The analytical procedure is based on concentration of actinides from 1800 1 sea water samples by hydroxide precipitations. Neptunium is isolated by ion exchange, fluoride precipitation and extraction with TTA (thenoyltrifluoroacetone). As a radiochemical yield determinant 239 Np or 235 Np is used. Neptunium is electroplated onto stainless steel discs before alpha-spectrometry for about I0 days. The procedure allows for sequential separation of plutonium, americium, technetium and radiocaesium together with neptunium. The radiochemical yield for neptunium is only 20-50%, but the procedure has been applied with success on several samples contaminated with ²³⁷Np at fallout or close to fallout levels.

Introduction

Neptunium-237 has been released to the environment mainly from nuclear weapon tests and nuclear fuel reprocessing plants. The total world integrated deposition from nuclear weapon tests is estimated at 2500 kg and the $237_{\text{Np}}/239+240_{\text{Pu}}$ activity ratio at 0.0050, 1.2 . There is an increasing interest in 237 Np, because, together with 99 Tc, it is one of the most important radionuclides remaining after long term storage of nuclear waste.³

Due to analytical difficulties our knowledge of neptunium in the environment is limited. In the present work a method has been developed for determining 237 Np in environmental samples, with particular emphasis on largevolume samples of seawater. There are radiochemical methods described in the literature for the determination of 237 Np in environmental samples 4, 5, 6 some of them combined with neutron activation analysis⁷ or using mass $spectrometry²$, but none of these was applied to large-volume seawater samples contaminated with fallout or close to fallout levels of 237Np.

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Experimental

Reagents and Apparatus

All reagents used are of analytical grade. A O.5.M. Thenoyltrifluoroacetone (TTA) solution is prepared by dissolving 11.1 g in 100ml xylene. Dilute acids such as 10M HCI, 9M HCI, 4.5M HCl/0.1M HF mixture, IM HCl, 10M HNO3, and 0.2M H₂S0₄ are prepared by dilution of concentrated acids with H₂0. Other reagents used are La-carrier, $F\in SO_{4}$, 0.3M Na₂SO₄, saturated H3BO4 solution, saturated NaOH solution, ammonia, thymol blue indicator, lM NH4I and H₂0₂ (30%). Neptunium-239 (T_{1/2}=2.35 d) obtained from ²⁴³Am $(T_1/2=7950$ a) or 235 Np (T₁/ 2410 d) is used as the radiochemical yield determinant. Large precipitation tanks (1800 1), stainless steel discs, electrolysis vials, and an electrolysis apparatus are used for sample and source preparations.

Preparation of 239 Np-yield determinant

Neptunium-239 is the daughter product of 243_{Am} . Take about 5-10 Bq of 243 Am from a stock solution. Evaporate and dissolve in 10 ml 10M HCl. Prepare a small anion exchange column (0=0.5 cm. height=Scm, Dowex ix4, 100-200 mesh). Precondition the column with 15 ml 9M HCl. Add 1 ml IM NH4I to the 243 Am solution. Let the sample pass the column at a speed of about 1 ml min $^{-1}$. Wash the column with 20 ml 9M HCl. Collect the effluent and the wash, which will contain 243Am. After a week this solution can be newly recycled for the recovery of 239 Np. Elute 239 Np with 20 ml 0.5M HNO3. Evaporate the solution to 1-2 ml. Take aliquots from this solution and add to the samples to be processed.

Sample Preparation

Transfer 1000-2000 1 of seawater to a conical tank. Acidify to pH 1-2 with HCl and add the radiochemical yield determinant 239 Np (~ 2 Bq) or 235 Np (\sim 0.5 Bq). Also add other yield determinants such as 242 Pu, 243 Am, 99 Tc^m, and 134 Cs, if analysis of plutonium, americium and/or americium technetium or radiocaesium will be carried out on the same sample. Mix mechanically or by air bubbling for 1-2 hours. Precipitate actinides with mixed hydroxides and carbonates of calcium and magnesium (naturally present in sea water). By

adding NaOH solution until the sample has a pH of 9-10. The supernant will contain technetium. Collect the precipitate (the tank should have a tap at the bottom). Dissolve the precipitate with HCI and perform a reprecipitation with ammonia (pH=9-10). Discard the aqueous phase or combine it with the previous aqueous phase if analysis of 99 Tc and/or radiocaesium will be performed. Centrifuge and dry the precipitate.

Ion Exchange

Prepare a column with ion exchange resin ($0=1$ cm, height=15-20 cm, Dowex ix4 100-200 mesh). Precondition the column with 100 ml 9M HCI. Dissolve the sample in 10M HCl (200-300 ml) and a few drops of H_2O_2 during heating. Cool the sample, centrifuge and discard the remaining solids. Add 1 ml IM NH4I per 10 ml of 10M HCI used. Heat gently a few minutes. (This will reduce Np and PU to the +4 and +3 oxidation states respectively.) Let the sample pass the column at a speed of $2-3$ ml min⁻¹. Wash the column with 75 ml of 9M HCl. (The effluent and the wash will contain Pu, Am, Th.) Elute Np with 200 ml 4.5M HCl-0.1M HF. Add 20 mg of La to the Np-eluate and add HF until LaF₃ precipitate forms. Centrifuge and discard the aqueous layer, and wash the precipitate with distilled water.

Solvent Extraction

Dissolve the LAF_3 precipitate in 10 ml conc. HCl and 2 ml saturated H3BO4 solution. Heat and take to dryness. Dissolve the sample again in HCl and precipitate hydroxides with ammonia. Centrifuge and wash the precipitate. Dissolve the sample in $1M$ HCl (5 ml) and add about 100 mg FeSO₄ in order to keep Np in the +4 oxidation state. Transfer the sample to a separation funnel with 5 ml 1M HCl. Extract Np with 7 ml 0.5M TTA in xylene and shake for 2 minutes. After separation of the phases, wash the organic phase with 10 ml IM HCl. Back-extract Np with 2x7 ml 10M HN03, leaving any iron in the organic phase. Wash the aqueous phase with i0 ml xylene in another separation funnel.

Electrodeposition

The procedure is the same as has been described earlier for other actinides. 8

Briefly, neptunium is electrodeposited onto stainless steel discs (0=20 mm) mounted in polyethylene scintillation vials. Add 1 ml 0.3M Na₂SO₄ to the sample and evaporate to dryness on a hot plate. Add 0.3 ml conc. H_2SO_4 and heat gently. Add 4 ml H_20 and adjust pH to about 2-3 with ammonia, using thymol blue as indicator. Transfer the sample to the scintillation vial with 5 ml 0.2M H2S04. Adjust pH to 1.9-2.1 using indicator paper. Electrolyze at IA for 2 hours. Add 1 ml almonia just before switching off the current. Dismount and rinse the planchet with 1% ammonia solution and acetone.

Yield Determination

The radiochemical yield is determined by gamma-spectrometry (Ge detector) or beta-counting. By gamma-counting higher activities of yield determinant should be used due to the generally lower detector efficiency. For gamma-counting of 239 Np the 106 keV transition and its characteristic K-x-rays) from Pu are used.

Neptunium-235 is mainly emitting low-energy L-x-rays from uranium. A planar Ge detector is the most suitable devise. However, a conventional GM-beta-counter which is sensitive for these low-energy photons can also be used to advantage.

Measurement of 237Np

Neptunium-237 is measured by alpha-spectometry for about 10 days. (min alpha-energy emitted = 4.64 MeV, max alpha-energy emitted = 4.87 MeV). Correct for any 234 U present by subtracting the 237 Np peak with 1.14 times the 238_U (4.15, 4.20 MeV) peak. Calculate the activity using the actual detector efficiency.

Results and discussion

The method described is suitable for determining 237 Np in large-volume seawater samples. The major problem with a measurement of 237 Np by alphaspectrometry is the interference of 234 U. This problem is particularly important for seawater samples, which usually contain 44 mBql⁻¹ of ²³⁴U. The levels of 237 Np from global fallout can be estimated at about 6.5x10⁻⁵ mBql⁻¹, if it is assured that the concentration of 239 , 240 Pu is about 13x10⁻³ mBql⁻¹ and the integrated fallout activity ratio $(237_{Np}/239_{+}240$ Pu) is 0.0050.

In seawater from the Sellafield nuclear fuel reprocessing plant area, the 237Np concentration was 9 mBq 1^{-1} in 1984 with a 237Np/239+240Pu activity ratio of $0.40.7$

The distribution-factor, K_{D} , (ratio between radioactivity concentration per kg of dry suspended matter and the activity concentration per liter in sea water) is smaller for neptunium (10^3-10^4) than for plutonium (10^5-10^6) . 10 This means that the residence time for neptunium is longer, which would give a higher ratio to plutonium than what is found in integrated fallout.

Several steps in the analytical procedure prevent interference from uranium. In the column operation, neptunium is eluted by fluoride Complexation. This does not affect uranium, which mainly remains on the resin. Uranium is not carried in the LaF3 precipitation step and the extraction coefficient of U (+6) with TTA is only $3x10^{-5}$ compared to 10^{4} for Np (+4). In spite of this, for some samples fully satisfactory removal of uranium was not obtained. This makes the corrections too large and the results statistically poor.

Protactinium-231 also has one of its alpha-energies (4.73 MeV) in the same region as 237 Np. However, concentrations of 231 Pa are low in seawater $(4x10^{-3} \text{ mBq1}^{-1})$, and 231 Pa is also eliminated in the fluoride precipitation step.

If preconcentration of the samples is carried out on board a ship, a more long-lived isotope than 239 Np, such as 235 Np must be used as radiochemical yield determinant. If analysis of 241 Am is to be carried out on the same sample, the 239 Np contains traces of 243 Am which are large enough to disturb the accurate yield determination of americium. However, the ²³⁵Np yield determinant we used contained some 237 Np, for which correction is necessary. The activity concentration of $237Np$ present in the $235Np$ solution was determined by a similar procedure, as described, and using 239 Np as radiochemical yield determinant. As a result, in July 1985, we obtained an activity ratio 237 Np/235Np of 1.7x10⁻³. This ratio increases with time. A 235 Np standard containing negligible amounts of 237 Np is now available.

We have analyzed 16 samples from the Barents, Greenland, and North Seas, collected in 1985 and 1981, respectively. T9 three of the samples no yield

determinant was added, in order to assure that no foreign radioactive elements would influence the count rate in our GM-gas flow counter when the yield determination was made.

The results are shown in Table 1. The samples originating from the Barents and Greenland seas are expected to oontain slightly higher activities than representative for fallout levels, due to nuclear waste from European fuel reprocessing facilities. Our results, which show 237_{NP} levels within the expected ranges, demonstrate the possibility of employing the present method for such studies. However, the correction for 234 U was between 20-60% of the total counts in the 237Np region and for 2 samples the results became negative, including the correction for presence of 237 Np in the 235 Np yield determinant. One sample from 71°N gave as result a $^{237}Np/239+240pu$ activity

Table 1 Activity concentrations of $239+240$ Pu and 237 Np in surface water from the North, Barents and Greenland Seas (mBq m-3)

*Values for $^{239+240}$ Pu obtained from Kautsky 11 . One sample from 85-07-14, 79002'N, 0052'W failed both for Pu and Np.

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ratio much higher than expected. The final purified neptunium fraction could conveniently be used for other measurement techniques, such as neutron activation analysis or mass-spectrometry.

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